

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) INDIAN POINT, UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 1 2 4 7 1				PAGE (3) 1 OF 2		
TITLE (4) AUXILIARY FEEDWATER PUMP RELAYS																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 9	1 0	8 4	8 4	0 1	2	0 1	0 4	0 3	8 5					0 5 0 0 0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)														
POWER LEVEL (10)		20.402(b)				20.402(e)				80.73(a)(2)(iv)				73.71(b)		
0 1 0 1 0		20.402(a)(1)(i)				80.38(a)(1)				80.73(a)(2)(v)				73.71(e)		
		20.402(a)(1)(ii)				80.38(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 355A)		
		20.402(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)						
		20.402(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)						
		20.402(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(ii)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME MICHAEL BLATT, REGULATORY AFFAIRS										TELEPHONE NUMBER						
										AREA CODE						
										9 1 4		5 2 6 5 1 2 7				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
X	BIA	IRILY	W11210	Y												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 10, 1984 when attempting to fill the steam generators utilizing the motor driven auxiliary feedwater pumps it was found that the pumps would not start. The plant was at cold shutdown for a refueling-maintenance outage when the event occurred. The cause of this event was determined to be two defective relay coils; both coils were open-circuited and the insulation decomposed. Although the coil failures were indicative of over-temperature, records verified that coil voltage, when energized, was maintained within the supplier's limitations.

8504160016 850403
PDR ADOCK 05000247
S PDRIE 22
11

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (8)

PAGE (3)

INDIAN POINT, UNIT 2

0 5 0 0 0 1 2 4 7 8 4 - 0 1 2 - 0 1 0 2 OF 2

TEXT (if more space is required, use additional NRC Form 366A (17))

On September 10, 1984, while at cold shutdown for a refueling-maintenance outage, the auxiliary feedwater pumps did not start when actuated. The pumps were being used for steam generator fill. The cause of the failure of the pumps to not start was traced to two relays, one per pump. Examination of the relays revealed open circuiting and severe degradation of the insulation. In the two week period prior to this event, three other similar relays failed with evidence of similar degradation.

When the plant is at power, these relays are usually energized. When at cold shutdown the relays may be energized.

This class of relays has experienced occasional past failures in applications where they are frequently energized during operation. The cause of previous failures was traceable to sensitivity of the relays to temperature. For replacement, relays coil insulation was modified and ambient temperature and voltage restrictions were recommended by the supplier.

The two failed relays of this event are frequently energized. During this outage, when the relays were energized, a check of operating voltage indicated it to be below the maximum recommended by the supplier. The cause of the failures is attributed to length of service. To preclude further failures an evaluation of similar relays is in progress with selective replacement. The supplier has been contacted and is assisting in this program.

There was no safety hazard due to these failures since the plant was in cold shutdown. In addition, the diverse steam driven auxiliary feedwater supply was unaffected by this condition.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

April 3, 1985

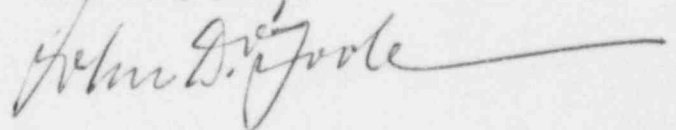
Re: Indian Point Unit No. 2
Docket No. 50-247
LER 84-012-01

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

Transmitted herewith is a revised report for Licensee Event Report LER 84-012-01.

Very truly yours,



attach.

cc: Dr. Thomas E. Murley,
Regional Administrator - Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

IE22
11