

UNITED STATES NUCLEAR REGULATORY COMMISSIONCOMMONWEALTH EDISON COMPANYDOCKET NOS. 50-373 AND 50-374NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO  
FACILITY OPERATING LICENSES, PROPOSED NO SIGNIFICANT HAZARDS  
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF-11 and NPF-18 issued to Commonwealth Edison Company (ComEd, the licensee) for operation of the LaSalle County Station, Units 1 and 2, located in LaSalle County, Illinois.

The proposed amendment would change the setpoints for the automatic primary containment isolation signal upon detection of a high main steamline tunnel differential temperature. Additionally, the proposed amendments would delete the automatic isolation function upon detection of a high main steamline tunnel temperature. Both these temperature generated signals detect possible steam leaks in the main steamline tunnel and initiate the isolation signals cited above, thereby providing automatic closure of the main steamline isolation valves (MSIVs) and the main steamline drain isolation valves. The intent of the proposed actions is to minimize spurious isolation signals which, in turn, would trip the reactor. The licensee proposes to provide for early detection of a main steamline break by relying on an automatic isolation signal which would be generated by a main steamline leak of 100 gallons per

minute (gpm) or greater. The current isolation setpoints are based on a steam leakage of 25 gpm in the main steamline tunnel.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

- 1) Involve a significant increase in the probability or consequences of an accident previously evaluated because:
  - a. There is no effect on accident initiators so there is no change in probability of an accident. The accident analysis associated with a steam line break in the main steam line tunnel assumes an instantaneous circumferential break of a main steam line downstream of the outermost isolation valve. The leak detection isolation on differential temperature based on less than or equal to 10 percent of a calculated critical crack of a main steam line is only a precursor of a break, and thus does not affect the probability of a break.
  - b. There is no or minimal effect on the consequences of analyzed accidents due to deletion of the automatic isolation on high temperature leak detection in the main steam line tunnel or due to increasing the leak detection differential temperature setpoint and

allowable values to detect a 100 gpm steam leak from a crack in a main steam line. The worst case accident corresponding to main steam lines outside of the reactor vessel and primary containment boundary is a main steam line break, which bounds the dose consequences of any size steam leak less than a full break. Also, a 200 gpm steam leak results in a calculated offsite dose within the annual whole body dose limit and the radioiodine release limit per 10CFR50 Appendix I, if detected and isolated within several weeks.

- 2) Create the possibility of a new or different kind of accident from any accident previously evaluated because:

The purpose of the main steam line isolation is based on leak detection and automatic isolation for leakage in the main steam line tunnel downstream of the outermost isolation valve. This change maintains this capability with only the leak detection based on high differential temperature in the steam line tunnel. Also, the primary containment isolation logic for main steam line leak detection isolation on high differential temperature remains the same. Thus no new or different accident is created.

- 3) Involve a significant reduction in the margin of safety because:

The increased setpoint for differential temperature leak detection for automatic isolation of the main steam lines due to a steam leak outside of the primary containment is based on calculated/analyzed response to a steam leak [that is] small compared to the leak from a critical crack. The leak detection isolation logic remains single failure proof. The previous evaluation of diversity of isolation parameters considered the ambient temperature and differential temperature isolations as one parameter in Table 5.2-8 of the LaSalle [Updated Final Safety Analysis Report] UFSAR. The deletion of leak detection isolation of the main steam lines based on high ambient temperature in the main steam line tunnel is acceptable, because the differential temperature isolation has been analyzed to detect and isolate the main steam lines based on bounding inlet air temperatures. Therefore, the Main Steam Line High flow, vessel low level, and the differential temperature instruments maintain adequate diversity of isolation parameters without main steam line tunnel high temperature.

The differential temperature leak detection for the main steam line tunnel depends on normal ventilation flow to

detect leakage. Therefore, the trip function will be declared inoperable upon loss of or shutdown of normal ventilation. The Technical Specifications currently allow the main steam tunnel high temperature and high differential temperature isolation channels to be inoperable for up to 4 or 12 hours during the performance of specified required surveillances. The 12 hours allowed outage time is currently for an 18 month surveillance requirement. The addition of allowance for up to 12 hours allowed outage time to recover normal ventilation following an unplanned loss of normal ventilation is reasonable, since the time is small compared to the time frame over which a pipe crack grows. Also, supplemental monitoring of water collection sumps and area temperature in the main steam line tunnel provides heightened awareness of operators to detect leakage in the main steam line tunnel during the time normal ventilation is not available. The planned shutdown of normal ventilation is currently allowed for up to 4 hours by the Technical Specifications. The unplanned loss of normal ventilation is expected to be less than two times per cycle upon completion [of] design changes to make the isolation logic power supply D.C. instead of A.C. through motor generator sets.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves

no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By March 28, 1996, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of

10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Jacobs Memorial Library, Illinois Valley Community College, Oglesby, Illinois 61348. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Robert A. Capra: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60603, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be

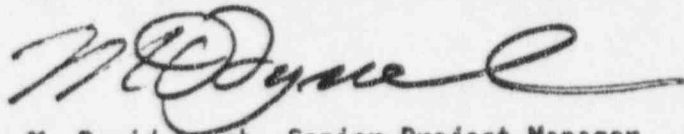


entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(1)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated January 18, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Jacobs Memorial Library, Illinois Valley Community College, Oglesby, Illinois 61348.

Dated at Rockville, Maryland, this 21th day of February 1996.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "M. David Lynch", written in black ink.

M. David Lynch, Senior Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation