

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 2 6 0										PAGE (3) 1 OF 2		
TITLE (4) Unplanned Initiation of Reactor Protection System During Shutdown																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)										
0	2	7	8	5	8	5	0	0	3	0	0	0	3	2	9	8	5	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																				
N		20.402(b)				20.406(c)				X 50.73(a)(2)(iv)				73.71(b)								
POWER LEVEL (10)		0 0 0				20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)				
		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 306A)								
		20.406(a)(1)(iii)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(A)												
		20.406(a)(1)(iv)				50.73(a)(2)(iii)				50.73(a)(2)(viii)(B)												
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)												
LICENSEE CONTACT FOR THIS LER (12)																						
NAME Pat N. Ebersole										TELEPHONE NUMBER 2 0 5 7 2 9 - 3 7 8 8												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS								
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR								
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

During the cycle 5 refueling outage for unit 2, an automatic scram occurred during a modification to the Reactor Protection System. All fuel had been previously removed from the reactor vessel. Following scram initiation, the reactor operator determined the cause of the scram and took actions necessary to reset the scram. The procedure controlling the work was deficient in that the instruction did not alert the operator that a scram would occur.

8504080595 850329
PDR ADOCK 05000260
S PDR

IE22
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1) Browns Ferry - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 6 0 8 5 -	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		0 0 3	0 0 3	0 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

At the time of this event, unit 2 was in its cycle 5 refuel outage with all fuel unloaded from the vessel, unit 1 was operating at 95 percent, and unit 3 was operating at 100 percent. This event affected unit 2 only.

During performance of a modification to the reactor protection system (RPS), an automatic scram occurred when fuses were removed from the circuitry in the RPS. After determining the cause of the scram, the unit operator took actions necessary to reset the scram.

The workplan used to control the modification of the RPS was deficient in that there was no step to alert the reactor operator that a reactor scram would occur when the step in the workplan to remove the fuses was performed. This should have been identified since an automatic scram is expected when these RPS fuses are pulled. In this sense, the scram initiation was unexpected and not part of a preplanned sequence.

Following scram initiation, all work that was being performed in relation to this workplan was immediately stopped. Before work was resumed, this workplan was revised to include steps necessary to ensure that the reactor operator would be alerted to the fact that activities were in progress that were going to cause initiation of a scram signal.

This event had no safety significance since the reactor was in cold shutdown, and all fuel had been removed from the reactor.

Responsible Plant Section -

Previous Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

March 29, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

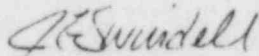
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/85003

The enclosed report provides details concerning the unplanned initiation of reactor protection system during shutdown. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Resident Inspector, BFN

IE22
1/1