

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) PALISADES NUCLEAR PLANT										DOCKET NUMBER (2) 0 5 0 0 0 2 5 5 1 OF 0 2										PAGE 13																					
TITLE (4) Reactor Trip																																									
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																							
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)								
																											NA						0 5 0 0 0								
0 8			0 4			8 4			8 4			0 1			5 0			1 1			1 3			0 8			4			NA						0 5 0 0 0					
OPERATING MODE (9)				THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																					
N																																									
POWER LEVEL (10)				0 4 8																																					
				20.402(b)																																					
				20.405(a)(1)(i)																																					
				20.405(a)(1)(ii)																																					
				20.405(a)(1)(iii)																																					
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				20.405(a)(1)(xx)																																					
LICENSEE CONTACT FOR THIS LER (12)																																									
NAME																		TELEPHONE NUMBER																							
David W. Rogers; Technical Engineer; Palisades																		6 1 6 7 6 4 1 8 9 1 3																							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																									
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC				CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																					
X		E/B		B/K/R		A/L/S/O		Y																																	
SUPPLEMENTAL REPORT EXPECTED (14)																																									
YES (If yes, complete EXPECTED SUBMISSION DATE)																		X		NO		EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR													
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																									
<p>On August 4, 1984, loss of electro hydraulic control (EHC) fluid pressure resulted in a turbine trip, and an automatic reactor trip. The Reactor Protection System (RPS) functioned as designed to shut down the reactor. No threat to public health or safety resulted.</p>																																									

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
PALISADES NUCLEAR PLANT	0 5 0 0 0 2 5 5 8 4	—	0 1 5	—	0 1	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On August 4, 1984, at 1355, a turbine [TRB;TA] trip occurred, resulting in an automatic reactor [RCT;AB] trip due to loss of load. The Plant was operating at approximately 48% power at the time of occurrence. No threat to public health or safety resulted.

The turbine trip was attributed to a loss of electro hydraulic control (EHC) fluid pressure, which allowed all major turbine [V;TA] operation valves to close. Subsequent investigation determined that a fitting on the discharge line from EHC Pump P-19A [P;TG] had backed completely off, resulting in a loss of EHC fluid inventory, as the fluid was pumped directly out of the system through the open discharge line. The fitting had worked loose as the result of excessive system vibration. A bracket which provides rigid support to the discharge line was noted to be missing. The bracket had been removed during the 1983/1984 refueling outage and was inadvertently not replaced during system reassembly. The root cause of the incident, therefore, was the failure of the Company's traveling maintenance crew to reinstall the support bracket on the EHC discharge line. The bracket was subsequently reattached in the appropriate location. The occurrence and its significance will be reviewed with management personnel from the responsible group.

During the incident, the Reactor Protection System functioned as designed to automatically shut down the reactor. The magnitude of the transient resulting from a reactor trip varies with power level. A reactor trip from full power, however, is an analyzed occurrence which would not place the Plant in a condition which is outside of its design basis. Additionally, Safeguards Bus 1-C [BU;EB] did not fast transfer to start-up power, but was picked up by Emergency Diesel Generator 1-1 [DG;EK]. Although a blown fuse [FU;EK] was later found in the undervoltage feature of Bus 1-C Supply Breaker 152-106 [BRK;EB], investigation indicates that it would not have been responsible for the failure of Bus 1-C to fast transfer to start-up power. Further investigation was conducted; however, the cause remains unknown. A test was subsequently performed which demonstrated a successful fast transfer of Bus 1-C to start-up power.



Consumers
Power
Company

November 30, 1984

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-015, REVISION 1 -
(REACTOR TRIP)

Attached please find Licensee Event Report 84-015, Revision 1 (Reactor Trip)
which is reportable to the NRC per 10 CFR 50.73(a)(2)(iv).

Brian D Johnson
Staff Licensing Engineer

CC Director, Office of Nuclear Reactor Regulation
Director, Office of Inspection and Enforcement
NRC Resident Inspector - Palisades

Attachment

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