

NOTICE OF VIOLATION

Northern States Power Company
Prairie Island Nuclear Plant

Docket Nos. 50-282; 50-306; 72-10
License Nos. DPR-42; DPR-60; SNM-2506

During an NRC inspection conducted on November 21, 1995, through January 2, 1996, three violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," NUREG-1600, the violations are listed below:

1. 10 CFR Part 72.48(b)(1) requires, in part, that the licensee maintain records of changes in the Independent Spent Fuel Storage Installation (ISFSI) if these changes constitute changes in the ISFSI as described in the Safety Analysis Report. These records must include a written safety evaluation that provides the bases for the determination that the change does not involve an unreviewed safety question.

Contrary to the above, on December 13, 1995, the lid was bolted and torqued on ISFSI cask TN-40-02, under License No. SNM-2506, after the cask had been loaded with irradiated fuel, and there was no written safety evaluation documented for a change in the ISFSI as described in the Safety Analysis Report involving the lid bolting.

This is a Severity Level IV violation (Supplement I).

2. 10 CFR Part 50.59(a) permits, in part, licensees to conduct tests or experiments not described in the safety evaluation report, without prior Commission approval, unless the proposed test or experiment involves a written safety question. 10 CFR 50.59(b)(1) requires, in part, that the licensee maintain records of tests and experiments carried out pursuant to paragraph (a). It further requires that the records include a written safety evaluation which provides the bases for the determination that the test or experiment does not involve an unreviewed safety question.

Contrary to the above, on November 15, 1995, the licensee performed a test of the emergency intake line, a test not described in the safety analysis report, without performing a written safety evaluation to determine if an unreviewed safety question was involved.

This is a Severity Level IV violation (Supplement I).

3. 10 CFR Part 50, Appendix B, Criterion III "Design Control" requires, in part, that measures be established to verify the adequacy of the design.

Contrary to the above, on May 8, 1995, the established verification measures failed to ensure the adequacy of the design described in calculation ENG-ME-147. Specifically, longitudinal stresses were calculated for internal pressure only and failed to evaluate the stress from bending due to unbalanced forces on the piping system, and the support evaluation methodology was invalid.

This is a Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Northern States Power Company is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. However, if you find it necessary to include such information, you should clearly indicate the specific information that you desire not to be placed in the PDR, and provide the legal basis to support your request for withholding the information from the public.

Dated at Lisle, Illinois,
this 6th day of February 1996