NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
North Anna Power Station	0 5 0 0 0 3 3 8	1 OF 0 2

TITLE (4)
Unit 1 Reactor Trip, December 31, 1984

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
1 2	1 8	8 4	8 4	— 0 2 6	— 0 1	0 3	2 5	8 5		0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(a)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
E. Wayne Harrell	7 0 3 8 9 4 - 5 1 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	A A	R J X	W 1 2 0	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

On December 31, 1984, at 0704 a Unit 1 reactor trip/turbine trip occurred from 100% power. The trip was caused by a failed firing card in the Rod Control System (EIIS system identifier AA) that allowed four control rods to drop into the core. The dropped control rods caused a negative flux rate reactor trip. All plant parameters responded normally.

While troubleshooting the Rod Control System with all rods withdrawn except the "D" control bank, group 1 of the "B" shutdown bank was dropped. The reactor was tripped manually while in mode 3. The failed firing card was replaced, a reactor start-up was commenced and criticality was reached at 1650 on December 31, 1984, without incident.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
North Anna Power Station	0 5 0 0 0 3 3 8	8 4	— 0 2 6	— 0 1	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On December 31, 1984, at 0704 a Unit 1 reactor trip/turbine trip occurred from 100% power. The trip was caused by a failed firing card in the Rod Control System (EIIS system identifier AA) that allowed four control rods to drop into the core. The dropped control rods caused a negative flux rate reactor trip. All plant parameters responded normally. This event is reportable pursuant to the requirements of 10CFR50.73 (a)(2)(iv) as an event that resulted in the automatic actuation of the Reactor Protection System.

While troubleshooting the Rod Control System with all rods withdrawn except the "D" control bank, group 1 of the "B" shutdown bank was dropped. The reactor was tripped manually while in mode 3. The failed firing card was replaced, a reactor start-up was commenced and criticality was reached at 1650 on December 31, 1984, without incident.

Westinghouse technical representatives were consulted to determine if a generic consideration existed. Westinghouse concurred that this was an isolated failure.

NORTH ANNA POWER STATION
P.O. BOX 402
MINERAL, VIRGINIA 23117

VIRGINIA POWER

March 25, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No. N-84-038A
NO/DAH: 11
Docket No. 50-338

License No. NPF-4

Dear Sirs:

The Virginia Power Company hereby submits the following Update License Event Report applicable to North Anna Unit No. 1. Information inadvertently omitted from LER block 11 is included.

Report No. LER 84-026-01

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,

E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Dr. J. Nelson Grace, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

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