

January 30, 1996

Mr. C. Lance Terry  
Group Vice President, Nuclear  
TU Electric  
Energy Plaza  
1601 Bryan Street, 12th Floor  
Dallas, TX 75201-3411

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON COMANCHE PEAK STEAM ELECTRIC  
STATION (CPSES) LICENSE AMENDMENT REQUEST (LAR) 95-008 UNIT 2 RELOAD  
ANALYSES AND UNIT 1 REACTOR COOLANT FLOW (TAC NOS. M94167 AND  
M94204)

Dear Mr. Terry:

Based on our review of the Texas Utilities Electric LAR 95-008 (TXX-95288)  
dated November 21, 1995, the NRC staff has identified additional information  
needed to continue its review. The detailed request for additional  
information (RAI) is discussed in the enclosure.

We request your response to our RAI within 30 days of receipt of this letter.  
If you have any questions, please call me at (301) 415-1038. This requirement  
affects nine or fewer respondents and, therefore, is not subject to the Office  
of Management and Budget review under P.L. 96-511.

Sincerely,

ORIGINAL SIGNED BY:

Timothy J. Polich, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosure: Request for Additional Information

cc w/encl: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Group Vice President, Nuclear  
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Sincerely,

A handwritten signature in cursive script, appearing to read "Timothy J. Polich".

Timothy J. Polich, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosure: Request for Additional Information

cc w/encl: See next page

Mr. C. Lance Terry  
TU Electric Company

Comanche Peak, Units 1 and 2

cc:  
Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
P. O. Box 1029  
Granbury, TX 76048

Honorable Dale McPherson  
County Judge  
P. O. Box 851  
Glen Rose, TX 76043

Regional Administrator, Region IV  
U.S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 400  
Arlington, TX 76011

Office of the Governor  
ATTN: Susan Rieff, Director  
Environmental Policy  
P. O. Box 12428  
Austin, TX 78711

Mrs. Juanita Ellis, President  
Citizens Association for Sound Energy  
1426 South Polk  
Dallas, TX 75224

Arthur C. Tate, Director  
Division of Compliance & Inspection  
Bureau of Radiation Control  
Texas Department of Health  
1100 West 49th Street  
Austin, TX 78756-3189

Mr. Roger D. Walker, Manager  
Regulatory Affairs for Nuclear  
Engineering Organization  
Texas Utilities Electric Company  
1601 Bryan Street, 12th Floor  
Dallas, TX 75201-3411

Texas Utilities Electric Company  
c/o Bethesda Licensing  
3 Metro Center, Suite 610  
Bethesda, MD 20814

George L. Edgar, Esq.  
Morgan, Lewis & Bockius  
1800 M Street, N.W.  
Washington, DC 20036-5869

COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2

DOCKET NOS. 50-445 AND 50-446

REQUEST FOR ADDITIONAL INFORMATION REGARDING

LICENSE AMENDMENT REQUEST 95-008

1. You have discussed the use of different co-resident fuel assembly designs in reference 1 (page 1 of 13, Attachment 2). Please provide the reference for the method that has been used for the core reload with mixed fuel for CPSES Unit 2, Cycle 3. Have all the provisions from the reference been satisfied such as that required for the analysis for the effect of stress from seismic forces between the different fuel types (Siemens and Westinghouse) and the DNBR penalty factors required for transition cores?
2. You have discussed meeting the minimum measured flow requirement in Technical Specification (TS) 3.2.5c in reference 1 (page 6 of 21, Attachment 2). Will this reload incorporate low leakage core loading? If so, this type of loading has resulted in increased hot streaming in many plants that has resulted in reduced indicated Reactor Coolant System (RCS) flow rates. Has this low leakage core loading effect been factored into the uncertainty analysis? Will this reduced indicated RCS flow be a problem for CPSES Unit 2, Cycle 3? Please provide the total flow rates in gpm measured from the calorimetric heat balance for the current cycles for Units 1 and 2. Also, please provide the references that approved the 1.8% uncertainty for the flow measurement and the 0.5% for the effects of lower plenum flow anomaly mentioned on page 3 of 13, Attachment 2.
3. Please provide the reference for the approved method used for obtaining the Overtemperature N-16 reactor trip setpoint for obtaining the total uncertainty as discussed in reference 1 (pages 1, 2, and 3 of 13, Attachment 2 and TS Table 2.2-1) and the Overpower N-16 trip setpoint (page 5 of 13, Attachment 2 and TS Table 2.2-1).
4. Please explain the difference between how the power is calculated using the N-16 power indication and that from the calorimetric power indication as discussed in reference 1 (page 11 of 21, Attachment 2).
5. Please provide a list of the NRC approved codes, with the titles of the approved reports, used for the Unit 2, Cycle 3 reload analysis.

ENCLOSURE

6. You mention on page 11 of 13 of Attachment 2 that the most relevant design basis analysis in Chapter 15 of the CPSES Final Safety Analysis Report (FSAR) which is affected by the change in the safety analysis value for the CPSES Unit 2 Overtemperature N-16 reactor trip setpoint is the Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power (FSAR section 15.4.2) and that all acceptance criteria were satisfied. Please provide information on all the Chapter 15 accident analyses that were performed for CPSES Unit 2, cycle 3 and indicate what approved codes were used for each accident or transient and why the results were acceptable (i.e., met the DNBR requirement, met the pressure requirement, etc.).
7. Provide the input parameters for (power, pressure, temperature, flow, and power density) used to calculate DNBR and other Chapter 15 analyses for Unit 2 Cycle 3 and the resultant DNBR value.
8. Provide the uncertainty values and bases used in the statistical combination of uncertainties as required by the safety evaluation report that approved RXE-91-002, "Reactivity Anomaly Events Methodology," dated January 19, 1993.