

January 25, 1996

Mr. W. R. Campbell, Vice President
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

SUBJECT: RELIEF FROM AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME) CODE
REQUIREMENT TO PERFORM VISUAL EXAMINATION OF INTERIOR SURFACE OF
REACTOR PRESSURE VESSEL SUPPORT SKIRT - BRUNSWICK STEAM ELECTRIC
PLANT, UNITS 1 AND 2 (BSEP 95-0188) (TAC NOS. M92608 AND M92609)

Dear Mr. Campbell:

On May 18, 1995, the Carolina Power & Light Company (CP&L) requested relief from the provisions of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," for the Brunswick Steam Electric Plant, Units 1 and 2 (BSEP). Specifically, CP&L requested relief from performing visual examination (VT-3) of the interior surface of each reactor pressure vessel's support skirt. This request for relief was for the second 10-year inservice inspection interval, which began at BSEP in July 1986.

Based upon the information submitted by CP&L, the Nuclear Regulatory Commission (NRC) has concluded that the ASME Code required VT-3 of the reactor pressure vessel support skirt interior surface is impractical. Therefore, the relief is granted as requested, pursuant to 10 CFR 50.55a(g)6(i). In making this determination, the staff has considered the burden on the licensee that could result if the requirements were imposed. Furthermore the NRC has concluded that the ASME Code-required VT-3 of the exterior surface of the reactor pressure vessel support skirt in conjunction with ultrasonic examination of the reactor pressure vessel-to-reactor pressure vessel support skirt welds provide reasonable assurance of continued structural integrity.

The NRC staff's evaluation and conclusions are contained in the enclosed safety evaluation (SE).

Sincerely,
(Original Signed By B. Buckley for)
David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-325 and 50-324

Enclosure: Safety Evaluation

cc w/enclosure: See next page

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*See Previous Concurrence

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

SUBJECT: RELIEF FROM AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME) CODE REQUIREMENT TO PERFORM VISUAL EXAMINATION OF INTERIOR SURFACE OF REACTOR PRESSURE VESSEL SUPPORT SKIRT - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2 (BSEP 95-0188) (TAC NOS. M92608 AND M92609)

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The NRC staff's evaluation and conclusions are contained in the attached safety evaluation.

Sincerely,

A handwritten signature in black ink that reads "B. C. Buckley for".

David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-325 and 50-324
Enclosure: Safety Evaluation
cc w/enclosure: See next page

Mr. W. R. Campbell
Carolina Power & Light Company

Brunswick Steam Electric Plant
Units 1 and 2

cc:

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