



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 84 TO FACILITY OPERATING LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

INTRODUCTION AND BACKGROUND

In November 1980, the NRC staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements", which includes all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications (TSs) were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of TSs for all of these items in Generic Letter 83-37. Generic Letter 83-37 was issued to all Pressurized Water Reactor (PWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's TSs to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of TSs were found.

By letters dated May 5, 1982, and March 22, 1984, Toledo Edison Company (the licensee) submitted proposed TSs which address the request made in Generic Letter 83-37. This evaluation covers the following TMI Action Plan items:

1. Post-Accident Sampling (II.B.3)
2. Containment Pressure Monitor (II.F.1.4)
3. Containment Water Level Monitor (II.F.1.5)
4. Instrumentation for Detection of Inadequate Core Cooling (II.F.2)

EVALUATION

1. Post-Accident Sampling (II.B.3)

The guidance provided by Generic Letter 83-37 requested that an administrative program be established, implemented and maintained to ensure that the licensee has the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions. The Post-Accident Sampling System is not required to be operable at all times. Administrative procedures are to be established for returning inoperable instruments to operable status as soon as practicable.

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The licensee has provided a proposed revision to the TSs which is consistent with the guidelines provided in our Generic Letter 83-37. We conclude that the licensee has an acceptable TS for the Post-Accident Sampling System.

2. Containment Pressure Monitor (II.F.1.4)

The Davis-Besse containment has been provided with two supplementary channels for monitoring containment pressure following an accident. The licensee has proposed TSs that are consistent with the guidelines contained in Generic Letter 83-37. We conclude that the proposed TSs for containment pressure monitor are acceptable.

3. Containment Water Level Monitor (II.F.1.5)

Narrow range and wide range containment water level monitors provide the capability required by TMI Action Plan Item II.F.1.5. The TSs for Davis-Besse contain limiting conditions for operation and surveillance requirements that are consistent with the guidance contained in Generic Letter 83-37. We conclude that the proposed TSs for containment water level monitors are acceptable.

4. Instrumentation for Detection of Inadequate Core Cooling (II.F.2)

Generic Letter 83-37 provided guidance on TSs for the subcooling margin monitors, a reactor coolant inventory tracking system and core exit thermocouples. The licensee had existing subcooling margin monitor TSs in place and submitted proposed TSs for the core exit thermocouples and inventory tracking system. We have reviewed the proposed TSs and conclude that they are acceptable as they meet the intent of our guidance contained in Generic Letter 83-37.

ENVIRONMENTAL CONSIDERATION

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 13, 1985

The following NRC personnel contributed to this Safety Evaluation:
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