

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7	PAGE (3) 1 OF 0 2
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TITLE (4)
Missed Refueling Surveillances

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 2	1 5	8 5	8 5	0 0 7	0 0	0 3	1 4	8 5	N/A		0 5 0 0 0
									N/A		0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																						
POWER LEVEL (10) 0 0 0	20.402(b)	20.406(a)(1)(i)	20.406(a)(1)(ii)	20.406(a)(1)(iii)	20.406(a)(1)(iv)	20.406(a)(1)(v)	20.406(e)	80.38(a)(1)	80.38(a)(2)	80.73(a)(2)(i)	80.73(a)(2)(ii)	80.73(a)(2)(iii)	80.73(a)(2)(iv)	80.73(a)(2)(v)	80.73(a)(2)(vi)	80.73(a)(2)(vii)	80.73(a)(2)(viii)(A)	80.73(a)(2)(viii)(B)	80.73(a)(2)(ix)	73.71(b)	73.71(e)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)	
										X													

LICENSEE CONTACT FOR THIS LER (12)													
NAME Kevin Sykes								(X-523)		TELEPHONE NUMBER			
								AREA CODE 8 1 5		9 4 2 7 2 9 2 0			

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)														
CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPROS
A				N										

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO														

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During normal refueling outage routine weekly Operating Surveillances DOS 700-1, 2, 3 and 4 were not completed on a weekly basis as required by Technical Specifications. The surveillances were not completed for one weekly period. The safety significance was minimal as the reactor was in refueling and the surveillances missed were SRM/IRM rod blocks. With the reactor in refuel only one rod can be out of the core at any given period. The event was caused by Operator error as the surveillances were overlooked on the weekly schedule.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	— 0 0 7	— 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Weekly Operating Surveillances 700-1, SRM Inoperable Rod Block Test, 700-2, IRM Downscale Rod Block Functional Test, 700-3, SRM Detector Position Rod Block Functional Test, and 700-4, IRM Detector Position Rod Block Functional Test, were not completed the week of 2/3/85.

Surveillances 700-2, 3 and 4 fulfill rod block test requirements 5, 10 and 6 listed in Table 4.2.1 of the Technical Specifications. The required frequency for these tests is once every 7 days. These surveillances were previously conducted on 2/1/85 and were not completed again until 2/15/85, which constitutes a Technical Specification violation.

This error is attributable to miscommunication. The surveillances which were done 2/1/85 were mistakenly reported to have been done 2/5/85. Under Technical Specification guidelines, the surveillances were then due for next completion the week of 2/10/85. Therefore they were not done the week of 2/3/85.

As a corrective action, the individual responsible for the recording error was made aware of his mistake. In addition, a module on adherence to surveillance schedules will be added to the Operator training program. The safety significance is minimal because the reactor was in the refuel mode, obviating the need for IRM and SRM rod blocks.

At Dresden, surveillance intervals have been exceeded infrequently in past years; however this is a first reported occurrence for missed refueling surveillances.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

March 14, 1985

DJS Ltr #85-295

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #85-007-0, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(i)(B).

D. G. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

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