NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Callaway Plant Unit 1	0 5 0 0 0 4 8 3	1 OF 0 2

TITLE (4)
Inadvertent Engineered Safety Features Actuations

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 0	2 7	8 4	8 4	— 0 5 7	— 0 0	1 1	2 1	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 1 9	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Michael E. Taylor - Superintendent, Operations	AREA CODE 3 1 4 6 7 6 - 8 2 0 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S J	F C	W 1 2 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/27/84 a Reactor Trip, Feedwater Isolation (FWIS), Auxiliary Feedwater Actuation (AFAS), and Steam Generator Blowdown Isolation (SGBDIS) occurred as a result of a low level in steam generator (S/G) "A." The required ESF equipment performed as designed during the incident.

The low S/G level occurred during the transfer of feedwater flow from the Main Feedwater (MFW) Bypass Control Valve to the MFW Control Valve. A blown fuse left the MFW Control Valve closed while the bypass valve was being closed, thus resulting in a low S/G level.

The plant was stabilized via plant procedures and the fuse replaced. The pertinent feedwater system procedure was revised to prevent similar incidents.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time was the public health or safety threatened.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 0350 CDT on 10/27/84, a Reactor Trip, Feedwater Isolation (FWIS), Auxiliary Feedwater Actuation (AFAS), and Steam Generator Blowdown Isolation (SGBDIS) occurred as a result of a low level in steam generator (S/G) "A." The plant was in Mode 1 and at 19% Reactor Power at the time of the trip. The ESF equipment performed as designed.

General Operating Procedure OTG-ZZ-00003, Plant Startup Less Than or Equal to 5% to 20% Power, requires S/G feedwater flow to be transferred from the Main Feedwater (MFW) Supply Bypass Flow Control Valves to the MFW Supply Flow Control Valves via Normal Operating Procedure OTN-AE-00001, Feedwater System. Per OTN-AE-00001, Rev. 1, the closed MFW Control Valves were placed in automatic control and the MFW Bypass Control Valves were manually closed. With feedwater flow decreasing as the MFW Bypass Valves were closing, the MFW Control Valves automatically opened to provide feedwater flow.

On 10/27/84 the operators were in the process of transferring feedwater flow during a plant startup. A blown fuse in fuse block 1AEK06A de-energized air solenoids which kept MFW Flow Control Station AE-FK-0510 (Westinghouse Electric Corp., Model No. 7300 M/A) from automatically opening S/G "A" MFW Control Valve, AE-FCV-0510. S/G "A" MFW Bypass Control Valve AE-FCV-0550 had been approximately 60% closed without the automatic opening of AE-FCV-0510. S/G "A" level decreased, but AE-FCV-0550 could not be opened in a sufficient amount of time to maintain level in S/G "A." A low level in S/G "A" resulted and the Reactor Trip, FWIS, AFAS, and SGBDIS occurred at 0350.

Emergency Operating Procedures E-0, Reactor Trip or Safety Injection, and ES-0.1, Reactor Trip Recovery, were performed satisfactorily and the plant stabilized. The fuse was replaced, Reactor Trip Breakers closed, and normal feedwater restored. This plant startup was the first which required transferring flow to the MFW Control Valves. The time at which the fuse blew is unknown. The MFW Control Valves were operated successfully on 7/28/84 per Operations Surveillance Procedure OSP-AE-V02CS, MFW Control Valve Mode 5 Operability.

To prevent recurrence of this incident, OTN-AE-00001, Rev. 2, was issued on 11/13/84 to change the method of transferring feedwater flow to the MFW Control Valves. The MFW Control Valves are now manually opened while the MFW Bypass Control Valves automatically close to maintain S/G level. No further problems have been experienced similar to this incident.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time did this event pose a threat to the public health or safety.

Previous occurrences: none

UNION ELECTRIC COMPANY
CALLAWAY PLANT

MAILING ADDRESS:
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FULTON, MO. 65251

November 21, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-983

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-057-00
INADVERTENT ENGINEERED SAFETY FEATURES ACTUATIONS

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning inadvertent Engineered Safety Features actuations.

Andrew P. Neuhalfen

for S. E. Miltenberger
Manager, Callaway Plant

MET/WRR/JWK/drs
Enclosure

cc: Distribution attached

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cc distribution for ULNRC-983

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