AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date March 10,1985
Telephone 609-935-6000
Extension 4455

Completed by J. P. Ronafalvy

Month February 1985

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	1109	17	1092
2	1102	18	1107
3	1115	19	1112
4	1108	20	1108
5	1105	21	856
6	1100	22	964
7	1083	23	1101
8	1104	24	1106
9	1091	25	1103
10	1097	26	1104
11	1115	27	1104
12	1071	28	1101
13	1107	29	
14	1116	30	
15	1114	31	
16	1092		

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1/1

OPERATING DATA REPORT

Docket No. 50-272
Date March. 10, 1985
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	February 1985	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

		This Month	Year to Date	Cumulative
11.	Hours in Reporting Period	672	1416	67225
12.	No. of Hrs. Reactor was Critical	672	1398.6	37222.1
13.	Reactor Reserve Shutdown Hrs.	0	0	3088.4
14.	Hours Generator On-Line	672	1395.7	35554.3
15.	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	2220055	4602077	107370074
17.	Gross Elec. Energy Generated (MWH)	761150	1576540	35492590
18.	Net Elec. Energy Generated (MWH)	731713	1514577	33612559
19.	Unit Service Factor	100	98.6	52.8
20.	Unit Availability Factor	100	98.6	52.8
21.	Unit Capacity Factor (using MDC Net)	100.9	99.1	46.3
22.	Unit Capacity Factor (using DER Net)	99.9	98.1	45.8
23.	Unit Forced Outage Rate	0	1.4	32.4

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH February 1985

Completed by J.P. Ronafalvy

Docket No. 50-272
Unit Name Salem No.1
Date March 10,1985
Telephone 609-935-6000
Extension 4455

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
85-130	2-21	F	24.5	A	5	-	HB	VALVEX	Intercept Valves Turbine
85-136	2-22	F	9.0	A	5	-	HB	VALVEX	Intercept Valves Turbine

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH February 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: March 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1304	Chemical & Volume Control	Buttwelding of valves to the header making each of them a "one piece connection".
1EC-1447	Fresh Water	Connect one end of the former concrete plant 4" water supply line into the station potable water system and connect the other end, at No. 1 Well House, to lines feeding Nos. 901, 902 and other buildings.
1EC-1588	Cable Trays	Retag, repull various cables throughout the plant to match changes made to wiring diagrams prior to implementation of the DCR system.
1EC-1619B	Reactor Coolant-Wide Range RTD Recorders	Replace existing recorders with seismically qualified recorders. Revise cable designation numbers on wide range RTD cables to reflect vital channels.
1EC-1716	Environmental Qualification	Replace existing transmitter with environmentally qualified Rosemount 1153D model (PA-227, PA7461); (S.I. pump discharge pressure and PA-211 bit pressure.
1EC-1749	Aux Building Ventilation	Replace existing cooling coils of all pump room coolers with new cooling coils made of AL-6X tubes and copper tins.

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1775	Circulating Water	Replace the inlet expansion joints of Unit 1 Main Condensers, additionally replace the remaining outlet expansion joints not changed during the main condenser retubing in 1980.
1EC-1819	Radiation Monitoring	Relocate sample line connections for main steam line RMS from the MS131 valves to the MS130 valves.
1EC-1832	Steam Generator Blowdown	Upgrade piping and valves at the condenser via the following changes: 1) Replace sch. 80 carbon steel piping between GB1&5 valves and condenser with type A312 TP316, 2) Install isolating valves (Mark D-8) on both sides of control valve numbers 11-14GB185.
1EC-1843	Neutron Flux Monitoring	Install a non-safety related source range neutron flux monitor at the remote hot shutdown panel.
1EC-1874A	Service Water	Revise 8" SW emergency supply line to Aux Feedwater from #12 SW nuclear header; revise 4" SW supply line to room and oil coolers from #12 SW nuclear header; line or replace SS SW supply and return piping to #12 CC Hx; add break flanges in SW supply and return piping to #12 CC Hx.
1EC-1880	Incore Instrumentation	Restore bent thermocouple column A5.

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1904	Aux Feedwater	Relocate the 4 aux feedwater flow transmitter square root extractors and power feed switches to the Relay Room which is a benign environment area.
1EC-1939	Main Generator Stator Water Cooling	Rewire auxiliary scanner terminal rack in the Relay Room.
1ET-1996	Chilled/Service Water	Test application of Garlock "Turbo-Star" cartridge type mechanical seal in Chiller Condenser Recirc Pumps (#11 and #13 only)
1ET-2021	Main Generator	Monitor various generator parameters during start of generator/turbine.
1SC-0363	Condensate	Modify condensate heater drain and circulating water pump motors to allow filling of oil reservoirs. (#12 Condensate Pump only)
1SC-0636	Well #1	Remove temporary structure and install smaller permanent enclosure for production well #1.
1SC-0913	Circ Water Traveling Screens	Revise shear pin sprockets on circulating water traveling screens.
1ST-1243	Circ Water Bearing Lube Pumps	Change pumps to some other type pump, such as a submersible trash pump.
1SC-1312A	Containment Air Lock	Install an alarm system to warn personnel of inadvertant air lock pressurization in the event seals leak into the air lock.

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1MD-0087	Vital Heat Trace for CVCS	Vital heat trace for 1CV145 boric acid batch tank sample valve.

MAJOR PLANT MODIFICATIONS
REPORT MONTH FEBRUARY 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: March 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1304	This DCR increases conservatism and thereby reduces the possibility of future occurrence of cracks/breaks at the test, vent and drain connections. In addition, the section modulus of the connection will be significantly increased which will improve the resistance to undetected shock load. No unreviewed safety or environmental questions are involved.
1EC-1447	The installation of piping to bypass production well #1 does not affect any previously performed safety analyses. No unreviewed safety or environmental questions are involved.
1EC-1588	The intended function of the system remains the same. No unreviewed safety or environmental questions are involved.
1EC-1619b	This DCP provides for the redesignation of existing "non-vital" associated cables to vital channels. No cable rerouting or installation is involved. No unreviewed safety or environmental questions are involved.
1EC-1716	The replacement transmitters perform the same function as the original ones. No unreviewed safety or environmental questions are involved.
1EC-1749	The intended function of the system remains the same. No unreviewed safety or environmental questions are involved.
1EC-1775	The expansion joints to be installed do not perform any safety related function. No unreviewed safety or environmental questions are involved.
1EC-1819	The relocation of the Main Steam Line RMS Sample Lines will not affect any safety analyses previously done. The function for both affected systems remains unchanged. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1832	This DCR upgrades the piping material used on the Steam Generator Blowdown Lines. No unreviewed safety or environmental questions are involved.
1EC-1843	This DCR meets a commitment to Appendix R of 10CFR50. This installation of the source range neutron flux monitor at the remote shutdown panel is an enhancement of the capability of the station to safely shut down in the event of a fire emergency. No unreviewed safety or environmental questions are involved.
1EC-1874A	This DCR upgrades existing Service Water System piping. The new piping will prevent the build-up of marine growth on the pipe wall and will prevent corrosion. No unreviewed safety or environmental questions are involved.
1EC-1880	This DCR provides replacement of the seal housing assembly and the installation of thirteen new thermocouples. The new seal assembly is fabricated with the conoseal shoulder intentionally high to permit height adjustment through in place machining eliminating the concern that the proper height could not be regained if permanent bends exist in the column after restoration. Since no changes are to be made that would affect the system, previous safety and accident analysis do not change nor is the margin of safety reduced. If all thirteen thermocouples are lost, the quadrant minimum requirement of three (3) will not be violated. Three areas of safety concern were looked at in extensive detail. These areas were Post Accident Core Assessment, Subcooling Margin Monitoring, and Prediction of Rod Cluster Control Assembly Misalignment. No unreviewed safety or invironmental questions are involved.
1EC-1904	This change supports the requirements of NUREG 0737. The installation will meet all the requirements of seismic I. No unreviewed safety or environmental questions are involved.
1EC-1939	This DCR does not affect the safe shutdown characteristics of the plant. No unreviewed safety or environmental questions are involved.
1ET-1936	This test does not affect any presently performed safety analysis, nor does it create any new hazards. The basis for the Tech. Specs. remains unchanged. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1ET-2021	No safety systems are affected by this DCR. No unreviewed safety or environmental questions are involved.
1SC-0363	The change in the upper and lower bearing oil fill piping will not affect the operation of the motors. No unreviewed safety or environmental questions are involved.
1SC-0636	The demolition of well #1 structure, as well as the installation of the small temporary shed for one (1) booster pump does not affect any previously performed structural safety analysis. No unreviewed safety or environmental questions are involved.
1SC-0913	This system does not affect presently performed safety analyses nor does it create any new hazards. This modification will not alter any plant discharge or process. No unreviewed safety or environmental questions are involved.
1ST-1243	This system does not affect presently performed safety analyses nor does it create any new hazards. This modification will not alter any plant discharge or process. No unreviewed safety or environmental questions are involved.
1SC-1312A	The intended function of this system remains unaffected. This modification will not alter any plant discharge or process. No unreviewed safety or environmental questions are involved.
1MD-0087	This DCR will not affect any safety function of the system involved. No unreviewed safety or environmental questions ae involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
0099156539	SMD	1	13 CHARGING PUMP
			FAILURE DESCRIPTION: LINE GOING TO THE PUMP SEAL WATER TANK HAS A WELD LEAK
			CORRECTIVE ACTION: CUT AND REWELDED WELD
85-01-16-036-5	SMD	1	13CFCU
			FAILURE DESCRIPTION: SW LEAK IN SUPPLY HEADER
			CORRECTIVE ACTION: WELD REPAIRED
85-01-04-040-1	SMD	1	12 ACCUMULATOR LEVEL INDICATOR AND ALARM
			FAILURE DESCRIPTION: COMPARATOR TRIPPING OUT OF SPEC.
			CORRECTIVE ACTION: REPLACED CAPACITORS
0099162113	SMD	1	12SW10
			FAILURE DESCRIPTION: VALVE STEM IS DISCONNECTED FROM VALVE DIAPHRAGM
			CORRECTIVE ACTION: REPLACED VALVE AND GASKET
009901204-9	SIC	1	1TE-431A/B LOW LEVEL AMP.
			FAILURE DESCRIPTION: UNABLE TO CALIBRATE
			CORRECTIVE ACTION: REPLACED THE LOW LEVEL AMPLIFIER

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-10-28-038-8	SMD	1	1C DIESEL GENERATOR PRE-LUBE PUMP FAILURE DESCRIPTION: PUMP LEAKS CORRECTIVE ACTION: SEAL REPLACED
84-10-18-018-9	SMD	1	13 RCP SEAL/STANDPIPE FAILURE DESCRIPTION: SEAL IS LEAKING EXCESSIVELY CORRECTIVE ACTION: REMOVED .100" OF SHIM FROM #3 SEAL
84-07-25-863-61	SIC		#11 RCL DT/TAVG FAILURE DESCRIPTION: CHANNEL OUT OF SPEC CORRECTIVE ACTION: REPLACED CAPACITOR AND ISOLATOR; ADJUSTED TO SPEC.
943627	SMD	1	VALVE 11GB4 FAILURE DESCRIPTION: VALVE FAILED LEAK RATE TEST CORRECTIVE ACTION: REPLACED PLUG, SEAT, PACKING AND
00995688	SMD	1	16SW24 FAILURE DESCRIPTION: LEAKING CORRECTIVE ACTION: REPLACED DIAPHRAGM
009901592-7	SIC	1	1LC-934B/F #12 ACCUMULATOR LEVEL INDICATOR AND ALARM FAILURE DESCRIPTION: COMPARATOR TRIPPING OUT CORRECTIVE ACTION: REPLACED CAPACITORS AND ADJUSTED SETPOINTS

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
009910634-6	SIC	1	1B5-540B #14 S/G STEAM FLOW PROTECTION CHANNEL I
			FAILURE DESCRIPTION: CAPACITOR TRIPPING OUT OF SPEC.
			CORRECTIVE ACTION: REPLACED CAPACITORS AND ADJUSTED TO SPEC.
009911902-1	SIC	1	CONTAINMENT PRESSURE RECORDER
			FAILURE DESCRIPTION: WIDE RANGE INDICATES LOW (DOES NOT MATCH OTHER CHANNELS)
			CORRECTIVE ACTION: REPLACED SERVO AMPLIFIER
0099120097	SIC		SV1144 EMERGENCY CONTROL AIR COMPRESSOR
			FAILURE DESCRIPTION: BLOWING THROUGH TO ATMOSPHERE
			CORRECTIVE ACTION: REPLACED SOLENOID VALVE
04-11-23-015-5	SIC		1R41 APD UNIT
			FAILURE DESCRIPTION: PUMP HI/LO FLOW ALARM IS UP
			CORRECTIVE ACTION: REPLACED PUMP
0099121735	SIC	1	VALVE 1CC131 RCP THERMAL BARRIER FLOW
			FAILURE DESCRIPTION: NO LOW FLOW ALARM WITH VALVE CLOSED
			CORRECTIVE ACTION: REPLACED SWITCHES AND CALIBRATED

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
85-01-17-006-2	SIC	1	VALVE 1CV[illegible] (ROOM COOLER CONT. VALVE) FAILURE DESCRIPTION: VALVE WILL NOT CLOSE CORRECTIVE ACTION: REBUILT SOLENOID VALVE
85-01-18-066-1	SIC	1	VALVE 11SW24 FAILURE DESCRIPTION: AIR SUPPLY TO VALVE BROKEN OFF CORRECTIVE ACTION: REPLACED TUBING FROM SOLENOID VALVE TO 11SW24 DIAPHRAGM
84-12-31-073-0	SIC	1	13 S/G LEVEL CHANNEL IV FAILURE DESCRIPTION: CHANNEL DRIFTING CORRECTIVE ACTION: REPLACED LOOP AND RECALIBRATED
85-02-15-068-5	SMD	1	BAE HEAT TRACE - DORIC POINT 195 FAILURE DESCRIPTION: INCORRECT THERMOSTAT READING CORRECTIVE ACTION: REPLACED HEAT TRACE AND RESET PROBES
85-01-03-120-81	SIC		1R19C FAILURE DESCRIPTION: SPIKING INTO ALARM CORRECTIVE ACTION: CLEANED CONNECTOR

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-12-25-004-4		1	13 CHARGING PUMP
			FAILURE DESCRIPTION: LOSS OF SPEED CONTROL
			CORRECTIVE ACTION: FREED JAMMED LINKAGE ON MOTOR COUPLING
84-12-26-039-2		1	VALVE 1NT31
			FAILURE DESCRIPTION: VALVE IS NOT STROKING PROPERLY
			CORRECTIVE ACTION: INSTALLED NEW REGULATOR
85-01-19-159-1		1	#12 WASTE GAS COMPRESSOR
			FAILURE DESCRIPTION: VALVE 12WG10 WILL NOT OPEN UPON START OF COMPRESSOR
			CORRECTIVE ACTION: REPLACED MOTOR DIAPHRAGM
84-08-31-040-2	NCS	1	HANGER #1A-SIS-160
			FAILURE DESCRIPTON: SPRING CAN INOPERABLE
			CORRECTIVE ACTION: SPRING CAN REWORKED
85-02-066-9	SMD	1	BAE HEAT TRACE - DORIC POINT 195
			FAILURE DESCRIPTION: SECONDARY HEAT TRACE READS HIGH
			CORRECTIVE ACTION: REPLACED HEAT TRACE AND RESET PROBES

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
009901651-6	SIC	1	RVLIS
			FAILURE DESCRIPTION: MONITORS NOT FUNCTIONING PROPERLY
			CORRECTIVE ACTION: REPLACED POWER SUPPLIES
85-01-31-057-3	SIC	1	TS 632
			FAILURE DESCRIPTION: OUTPUT POSITION LIGHT TOOK TOO LONG TO ENERGIZE
			CORRECTIVE ACTION: CLEANED CONTACTS; INSTALLED NEW MICRO SWITCH; AND RESET LATCH COIL
85-01-17-004-6	SIC	1	VALVE 1SW205 (ROOM COOLER VALVE)
			FAILURE DESCRIPTION: VALVE DOES NOT SHUT OFF AUTOMATICALLY
			CORRECTIVE ACTION: SV-608 RE-ENERGIZED, REPLACED COIL
85-01-20-011-5	SMD	1	#12 AREA SPACE HEATER
			FAILURE DESCRIPTION: HEATING COIL IS NOT ENERGIZING
			CORRECTIVE ACTION: REPLACED SHORTING BAR
85-02-02-010-2	SMD	1	BREAKER FOR PRESSURIZER HEATERS
			FAILURE DESCRIPTION: BREAKER IS TRIPPING
			CORRECTIVE ACTION: REPLACED BREAKER

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
009911916-1	SIC	1	1R41B (PLANT VENT IODINE - RMS) FAILURE DESCRIPTION: MONITOR READOUT IS INDICATING A LARGE SWING IN READINGS WITH NO OTHER MONITOR INDICATIONS CORRECTIVE ACTION: REPLACED POWER SUPPLY MODULE
85-01-25-097-0	SIC	1	APD UNIT (R11A AND R12A) FAILURE DESCRIPTION: PUMP IS SEIZED CORRECTIVE ACTION: INSTALLED NEW VACUUM PUMP
942536	SIC	1	1R46A-D FAILURE DESCRIPTION: STEAM ISOLATION VALVES LEAKING THROUGH CORRECTIVE ACTION: REPLACED SHORTED COIL AND REBUILT VALVE (A); REPLACED BROKEN PISTON ASSEMBLY AND REBUILT VALVE(S) (B AND C); AND REPLACED CRACKED PLUNGER HOUSING AND REBUILT VALVES (D)

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
FEBRUARY 1985

SALEM NO. 1

The Unit began the period operating at full power. On 2/21/85 at 0140 hours, the Unit power was reduced to 79% as a result of 12E Turbine Intercept Valve failing to reopen after completion of a routine turbine valve surveillance test. The valve failed to reopen as a result of a false closure signal generated from the Electro-Hydraulic Control circuitry due to a failed resistor. On 2/22/85 at 1145 hours, with the completion of repairs, the Unit was returned to 100% power where it remained for the rest of the period.

REFUELING INFORMATION

DOCKET NO.: 50-272
COMPLETED BY: J. Ronafalvy
UNIT NAME: Salem 1
DATE: March 10, 1985
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month February 1985

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE _____

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

March 10, 1985

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of February 1985 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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