NRC Form 366 (9-63)	LIC	CENSEE EVENT RE		U.S. NUCLEAR HEQULATORY COMMISSION APPROVED (MS NO. 3150-0104 EXPIRES 8/31/85		
PACILITY NAME (1) OYSTER CREE!	K, UNIT 1		0 5 0 0	PAGE (E.		
TITLE (4)			CHEROMES N. C. L. C. C. C.			
		NSER PIPE BREAK	SENSORS OUT OF SPEC.			
	I I I I I I I I I I I I I I I I I I I		OTHER FACILITIES INV	DOCKET NUMBER(S)		
MONTH DAY YEAR	YEAR NUMBER NUMBER		PACILITY NAMES	0 15 10 10 10 1 1 1		
0 2 1 0 8 5	8 5 - 0 0 5 - 0 0	10 3 0 0 0 0 3		0 5 0 0 0 1		
OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT	TO THE REQUIREMENTS OF 10	CFR §: (Check one or more of the following) (50.73(a)(2)(iv)	72.71(b)		
POWER LEVEL (10) 0 0	20.406(a)(1)(i) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iv)	50.36(c)(1) 50.36(c)(2) 60.73(a)(2)(i) 50.73(a)(2)(ii)	50.73(a)(2)(vi) 50.73(a)(2)(vii)(A) 50.73(a)(2)(viii)(B)	73.71(a) OTHER (Specify in Abstract below and in Text, NRC Form 386.A)		
	20.405(a)(1)(v)	60.73(a)(2)(iii)	50.73(e)(2)(x)			
NAME		LICENSEE CONTACT FOR THIS	CER (12)	TELEPHONE NUMBER		
Michael G. Kar	oil, Senior Engineer		AREA CODE 6 0 9	9,7,1,4,8,9,1		
	COMPLETE ONE LINE FO	R EACH COMPONENT FAILURE	DESCRIBED IN THIS REPORT (13)			
CAUSE SYSTEM COMPO	NENT MANUFAC REPORTABLE TO NPROS	CAUSE	SYSTEM COMPONENT MANUFAC- TURER	REPORTABLE TO NPROS		
0 0 1 0 0	T C T 2 O A V					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten (ines) (16)

YES III yes, complete EXPECTED SUBMISSION DATE!

During routine surveillance testing, pipe break sensors IBO5A1, IBO5B1, IB11A1, IB11A2, IB11B1 and IB11B2 for both isolation condensers steam and condensate lines, tripped at values greater than specified in the technical specifications, Table 3.1.1.

Sensors IBO5A1, IBO5B1, IB11A2, IB11B1 and IB11B2 were reset to trip within desired set point limits. Sensor IB11A1 had a defective switch actuating cam; the defective cam was replaced and the sensor was set to trip within limits.

The event had no effect upon public health or safety.

SUPPLEMENTAL REPORT EXPECTED (14)

1027

MONTH

EXPECTED SUBMISSION DATE (15) DAY

YEAR

8503210109 850308 PDR ADOCK 05000219 S PDR NRC Form 386A

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)				PAGE (3)
			YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	
OYSTER CREEK, UNIT 1	0 5 0 0 0 2 1	9	815	_	0 10 15	-010	0 2 0 0 4

FEXT (If more space is required, use additional NRC Form 386/La) (17)

DATE OF OCCURRENCE

The event occurred on F bruary 10, 1985 at approximately 0330 hours.

IDENTIFICATION OF OCCURRENCE

During surveillance testing, isolation condenser pipe break sensors IBO5Al, IBO5Bl, IBl1Al, IBl1A2, IBl1Bl and IBl1B2 tripped at values greater than specified in the Technical Specification Table 3.1.1, Item H.

The event is considered to be reportable as defined in 10CFR50.73(a)(2)(i)(B).

CONDITIONS PRIOR TO OCCURRENCE

The Mode switch was in the shutdown position with reactor coolant temperature $\leq 212^{\circ}F$.

DESCRIPTION OF OCCURRENCE

While performing the test and calibration of the steam and condensate pipe break sensors in the isolation condenser system, the trip setpoints for six out of the eight sensors were found to be less conservative than permitted by the Technical Specifications. Surveillance testing of sensors yielded the following data:

Switch Designat			Tech Spec.	"As Found"	"As Left"
IB05A1	Steam Pipe, Cond.	A	≤ 20 psid	21 psid	15.5 psid
1805A2	Steam Pipe, Cond.	A	≤ 20 psid	20 psid	15.25 psid
IB05B1	Steam Pipe, Cond.	В	≤ 20 psid	21.5 psid	14.5 psid
IB05B2	Steam Pipe, Cond.	В	≤ 20 psid	18.4 psid	14.0 psid
IB11A1	Cond. Pipe, Cond.	Α	≤ 27 in. H ₂ 0	*60 in. H ₂ 0	**
IB11A2	Cond. Pipe, Cond.	Α	≤ 27 in. H ₂ 0	30 in H ₂ 0	25 in. H ₂ 0
181182	Cond. Pipe, Cond.	В	≤ 27 in. H ₂ 0	32.2 in. H ₂ 0	24.5 in. H ₂ 0
IB11B2	Cond. Pipe, Cond.	В	≤ 27 in. H ₂ 0	31.8 in. H ₂ 0	24.6 in. H ₂ 0

^{*}Test input pressure limited to sixty inches. With 60 inches applied, switch had still not tripped.

^{**} Due to malfunction of the sensor, technicians were not able to reset the switch within the required "As Left" tolerances. The switch actuating cam for this sensor was subsequently replaced.

U.S. NUCLEAR REGULATORY COMMISSION NRC Form 366A LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO 3150-0104 EXPIRES: 8/31/85 DOCKET NUMBER (2) FACILITY NAME (1) LER NUMBER (6) PAGE (3) SEQUENTIAL YEAR OYSTER CREEK UNIT 1 0 10 0 3 OF 014 815 010151-0 |5 |0 |0 |0 |2 |1 |9

TEXT (If move space is required, use additional NRC Form 366A's) (17)

APPARENT CAUSE OF OCCURRENCE

The cause of occurrence is attributed to instrument drift for sensors IBO5A1, IBO5B1, IB11A2, IB11B1, and IB11B2. Sensor IB11A1 was found with a defective switch actuating cam.

ANALYSIS OF OCCURRENCE

The isolation condenser pipe break sensors are designed to provide protection in the event of a steam or a condensate line break. Four pipe break sensors are installed in the piping of each emergency condenser; two sensors are for the detection of high flow in the steam line, and two are for the detection of high flow in the condensate line. Should one of these sensors detect a high flow condition lasting as long as 35 seconds, the isolation valves to that condenser are given a close signal.

SAFETY SIGNIFICANCE

Any one out of the four pipe break sensors (two in the condensate line, and two in the steam line) installed in each isolation condenser system, will, upon detecting a high flow, send a signal to isolate that Isolation Condenser System. A review of "As Found" sensor switch settings indicates that in the event of a pipe break, the steam line break sensor IBO5A2 (in Isolation Condenser System A), and steam line break sensor IBO5B2 (in Isolation Condenser System B), which were operating within the technical specification limits, would have actuated to isolate the affected Isolation Condenser System in the required manner.

Based on the above, the safety significance of this occurrence is considered minimal.

CORRECTIVE ACTION

Sensors IB05Al, IB05Bl, IB11A2, IB11Bl, and IB11B2 were reset to trip within the limits required by Technical Specifications. (Mote the "As Left" values in the description of occurrence.) The switch actuating cam for sensor IB11Al was replaced and the sensor was set to trip within the limits required by Technical Specifications. An inspection was performed on a sample of other sensors for any degradation. The Switch Actuating cams and mechanisms were found in satisfactory condition in these sensors.

Due to the frequency of setpoint drift problems with these snap-action type switch sensors, it is GPU Nuclear's intent to replace these sensors with ones having better accuracy and repeatability specifications, as part of the Reactor Protection System instrument upgrade, scheduled for the next (Cycle 11) refueling outage.

EQUIPMENT DATA

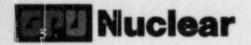
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Manufacturer - ITT Barton.

Model No. 288A indicating pressure switch.

Range: Steam line break sensors (IBO5's): 0-50 psid.

Condensate line break sensors (IBI1's): 0-60 inches H20.



GPU Nuclear Corporation

Post Office Box 388 Route 9 South Forked River, New Jersey 08731-0388 609 971-4000 Writer's Direct Dial Number:

March 8, 1985

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station

Docket No. 50-219 Licensee Event Report

This letter forwards one (1) copy of Licensee Event Report (LER) No. 85-005.

Very truly yours,

Peter 3. Fiedler

Vice President and Director

Oyster Creek

PBF:BH Enclosures

cc: Dr. Thomas E. Murley, Administrator Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

> NRC Resident Inspector Oyster Creek Nuclear Generating Station Forked River, NJ 08731

> > IEZZ