

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 1 3	PAGE (3) 1 OF 0 1 2
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TITLE (4)
Unplanned Actuation of Engineered Safety Features

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	0	2	1	8	4	8	4	4			0 5 0 0 0
											0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9) 2	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 0 5	20.406(a)(1)(i)	50.36(e)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	50.36(e)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Charles D. Naslund - Superintendent, I&C	TELEPHONE NUMBER
	AREA CODE 3 1 1 4 6 1 7 1 6 1 - 1 8 1 5 1 0 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/21/84 at 0518 CDT an unplanned Engineered Safety Feature actuated as a result of a hi-hi level signal on steam generator (S/G) 'A.' The plant was in Mode 2, 4.5% power, at the time of the event.

Technicians were in the process of calibrating a relay card LY/519 which provides a level signal to feedwater control valve FCV-510 and feedwater bypass control valve FCV-550. Personnel were unaware that LY/519 provided a level signal to FCV-550 which was in automatic control of feedwater to S/G 'A.' Thus when LY/519 was removed for calibration it also removed the level signal from FCV-550 causing it to fully open and fill S/G 'A' to the hi-hi setpoint level. This resulted in a Feedwater Isolation Signal, Steam Generator Blowdown Isolation Signal and Auxiliary Feedwater Actuation Signal.

The cause of this event was inadequate pre-planning of the work activity. A procedure revision has been developed which will better inform personnel as to the consequence of a work activity.

There was no damage to plant equipment or release of radioactivity as a result of this event. At no time was public health or safety threatened.

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PDR ADDCK 05000483
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		0 5 5	0 1 0	0 1 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 10/21/84 at 0518 CDT an unplanned Engineered Safety Feature Actuation occurred when a hi-hi level signal was received on steam generator 'A.' All components responded as designed with the plant in Mode 2, 4.5% reactor power, at the time of the event.

Technicians were attempting to calibrate a relay card LY/519 which provides a level signal to feedwater control valve FCV-510 and bypass control valve FCV-550. Neither the technician nor the operator recognized the necessity to place FCV-550 in manual control. Feedwater to steam generator 'A' was being automatically regulated through bypass control valve FCV-550. A technician proceeded to remove LY/519 for calibration. After approximately 3 minutes a hi-hi level signal was received on steam generator 'A.' This resulted in a Feedwater Isolation Signal (FWIS), Steam Generator Blowdown Isolation Signal (SGBIS) and an Auxiliary Feedwater Actuation Signal (AFAS). Systems returned to normal when LY/519 was replaced and by 0535 the FWIS and AFAS were reset.

The cause of this incident was inadequate pre-planning of the work activity. The technician and operators were unaware that LY/519 provided a level signal to FCV-550. Thus when LY/519 was removed with FCV-550 in automatic control, the loss of level signal caused FCV-550 to fully open and fill steam generator 'A' to the hi-hi setpoint level causing the FWIS, SGBIS and AFAS.

To prevent recurrence of this type of situation a revision to administrative procedure APA-ZZ-00320, "Initiating and Processing Work Requests," has been issued which provides for Planning Instructions as an attachment to work requests to better inform work groups and Operations personnel as to the consequence of a work activity. This revision was also discussed in LER 84-047-00.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time did this event pose a threat to the public health or safety.

Previous occurrences: LER 84-047-00

UNION ELECTRIC COMPANY
CALLAWAY PLANT

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November 20, 1984

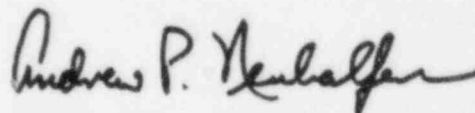
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-980

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-055-00
UNPLANNED ACTUATION OF ENGINEERED SAFETY FEATURES

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning an unplanned actuation of Engineered Safety Features.



for
S. E. Miltenberger
Manager, Callaway Plant

CDN/WRR/RCW/drs
Enclosure

cc: Distribution attached

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cc distribution for ULNRC-980

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N. Date

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