

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 230 License No. DPR-59

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated May 12, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications co tained in Appendices A and B, as revised through Amendment No. 250, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

TorLedyard B. Marsh, Director

Project Directorate I-1 Division of Reactor Projects - I/II

Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 21, 1995

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Revise Appendix A as follows:

Remove Pages 240 241 Insert Pages 240

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3.11 (cont'd)

D. Emergency Service Water System

 To ensure adequate equipment and area cooling, both ESW systems shall be operable when the requirements of specification 3.5.A and 3.5.B must be satisfied, except as specified below in specification 3.11.D.2. 4.11 (Cont'd)

D. Emergency Service Water System

 Surveillance of the ESW system shall be performed as follows:

		item	Frequency
	8.	Simulated Automatic Actuation Test	Once every 24 months
ŧ	b.	Flow Rate Test - Each Once/3 months ESW pump shall deliver at least 1500 gpm to its respective loop. The pump total developed head shall be greater than or equal to the corresponding point on the pump curve, reduced by a maximum of 7%, for the measured flow.	
	c.	Pump Operability	Once/month
	d.	Motor Operated Valves	Once/month

4.11 (cont'd)

e. ESW instrumentation check

Once/day

ESW instrument channel calibration

Once/3 months

f. Logic System Functional Test Once every 24 months

- From and after the time that one Emergency Service
 Water System is made or found to be inoperable for any
 reason continued reactor operation is permissible for a
 period not to exceed 7 days, provided that:
- testing.

2. ESW will not be supplied to RBCLC system during

- the operable Emergency Diesel Generator System is demonstrated to be operable immediately and daily thereafter; and,
- all Emergency Diesel Generator System emergency loads are verified operable immediately and daily thereafter.
- If specification 3.11.D.2 cannot be met the reactor shall be placed in the cold condition within 24 hours.
- 3. Not Used