

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1): LaSalle County Station Unit 2  
DOCKET NUMBER (2): 050000374  
PAGE (3): 1 OF 03

TITLE (4): APRM Hi-Hi Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
10	27	84	84	071	0	11	26	84			050000
<p>THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)</p>											

OPERATING MODE (9): 1	20.402(b)	20.405(c)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10): 0.99	20.405(a)(1)(ii)	80.36(c)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(iii)	80.36(c)(2)	<input type="checkbox"/>	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iv)	80.73(a)(2)(i)	<input type="checkbox"/>	80.73(a)(2)(vii)(A)	
	20.405(a)(1)(v)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(vii)(B)	
	20.405(a)(1)(vi)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: James J. Hietala  
TELEPHONE NUMBER: 815 357 1676

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	A/D	X F M R	S O 5 2	Y	X	A/D	C B L	S O 5 2	Y
X	A/D	X X X X	S O 5 2	Y					

SUPPLEMENTAL REPORT EXPECTED (14):  YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15): MONTH: | | DAY: | | YEAR: | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 27, 1984, at 0450, Unit 2 scrambled due to an Average Power Range Monitor "Hi-Hi" flux signal on Reactor Protection System channels A and B. The flux signal was caused by a vessel level transient, which occurred when the "B" Reactor Recirculation Flow Control Valve lost its position feedback and ramped open. The Unit was at 99% power (steady state) when the scram occurred. All systems responded as required. Because the problem was transient in nature, the problem could not be isolated to one component. The position feedback circuit components were replaced per Work Request L43044.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
LaSalle County Station Unit 2	05000374	84	071	00	02	OF 03

TEXT (If more space is required, use additional NRC Form 388A's) (17)

I. EVENT DESCRIPTION

On October 27, 1984, at 0450, Unit 2 scrambled due to an Average Power Range Monitor (IG) "Hi-Hi" flux signal on Reactor Protection System (JC) channels A and B. Just prior to the event, a transient in vessel level occurred, due to the "B" Reactor Recirculation (AD) Flow Control Valve (FCV) losing its position feedback and ramping open. All systems operated as expected after the "Hi-Hi" flux signals occurred.

II. CAUSE

The source for the event was an erroneous feedback signal of the "B" Reactor Recirculation Loop, Flow Control Valve position. The feedback signal is provided by a Linear Variable Differential Transformer (LVDT), fed through a cable to a signal conditioner. Because the erroneous signal was of a transitory nature, the cause could not be verified to any one of the possible components: LVDT, cable, or signal conditioner. The LVDT and signal conditioner are manufactured by Schaevitz.

The plant was at 99% power (steady state) when the event occurred. The alarm typer was used to verify the initial cause of the event. No Emergency Core Cooling System (ECCS) response was required, and none occurred. Primary Containment Isolation System (PCIS) Group 6 and 7 valves were closed prior to the event. No PCIS (JM) response was required, and none occurred. The Reactor Protection System operated as required during the event.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

All systems operated as designed. Minimum Reactor Level was -35 inches; minimum Reactor pressure was 860 pounds; maximum pressure after the decrease was 920 pounds. Safe plant operating conditions were maintained at all times.

IV. CORRECTIVE ACTION

Investigation of the possible components for failure did not reveal the component at fault. The components were all checked and found to work satisfactorily after the event. Therefore, the LVDT, the cable from the LVDT to a drywell junction box, and the signal conditioner were all replaced, per Work Request L43044.

V. PREVIOUS OCCURRENCES

There have been previous operational problems with the position feedback circuit, however, this is the first time a scram has occurred due to the feedback circuit problems.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR 8 4	SEQUENTIAL NUMBER 0 7 1	REVISION NUMBER 0 0			

TEXT (If more space is required, use additional NRC Form 306A's) (17)

VI. NAME AND TELEPHONE NUMBER OF PREPARER

James J. Hietala, 815/357-6761, extension 499.



**Commonwealth Edison**  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
Marseilles, Illinois 61341  
Telephone 815/357-6761

November 26, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-071-00, Docket #050-374 is being submitted to your office in accordance with 10CFR 50.73.

*G. J. Diederich*  
G. J. Diederich *11/26/84*  
Superintendent  
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director  
INPO-Records Center  
File/NRC

*TF22*  
*1/1*