

A-197

GPC Exhibit II-197
Hill/Ward Exhibit C



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W., SUITE 2000
ATLANTA, GEORGIA 30303-0100

DOCKETED
USNRC

August 14, 1995

'95 OCT 20 P4:45

Georgia Power Company
ATTN: Mr. C. K. McCoy
Vice President
Vogtle Electric Generating Plant
P. O. Box 1298
Birmingham, AL 35201

OFFICE OF SECRETARY
DOCKETING & SERVICE
BRANCH

SUBJECT: RESOLUTION OF OUTSTANDING ISSUE ASSOCIATED WITH CALIFORNIA
CONTROLS SWITCHES USED ON EMERGENCY DIESEL GENERATORS

Dear Mr. McCoy:

This refers to an outstanding issue discussed in our October 19, 1990, letter associated with your root cause analysis of the failure of the emergency diesel generator (EDG) to provide AC power as intended. This concern was precipitated by the past history of reliability of California Controls Company (Calcon) sensors. These sensors were used on the EDGs at the Vogtle facility and became a significant concern during the Site Area Emergency (SAE) that occurred on March 20, 1990.

An enforcement conference was held at our request on September 5, 1990, to discuss numerous items identified by the NRC Incident Investigation Team (IIT) which investigated the circumstances of the SAE. The primary items discussed at the enforcement conference were the failure to make timely emergency notifications to state and local government agencies, the inability of site personnel to establish containment integrity within the required time limits, and the failure of the EDG. The internal contamination found in the Calcon jacket water temperature sensors and inconsistent calibration techniques resulting in intermittent Calcon sensor failure were identified as the most probable cause of the EDG trips. In the October 19, 1990 letter, we informed you that the NRC concerns associated with your root cause analysis of EDG problems would be addressed separately.

The IIT report, NUREG 1410, was issued in June 1990. In this report, the IIT identified that a significant number of Calcon sensor failures had occurred at Vogtle since 1985. The NUREG identified a list of failures that had occurred between 1985 and 1990. In your letter dated July 9, 1990, documenting your review of the NUREG, you took exception to the conclusion reached by the IIT with respect to the sensor failures. In your review you pointed out that a large percentage of the problems identified were associated with calibration setpoints being out-of-specification during construction acceptance testing. You also identified that an out-of-calibration condition is not typically counted as a failure by either Georgia Power Company (GPC) or other plants in accordance with the Nuclear Plant Reliability Data System reporting criteria and, therefore, the conclusion reached by the NRC was not based on comparable data. The NRC reviewed this correspondence and did not disagree with the GPC position and a formal reply was not provided.

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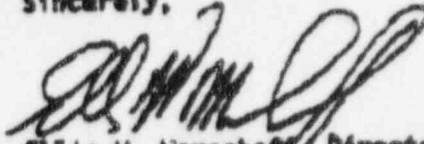
NUCLEAR REGULATORY COMMISSION
Docket No. 50-424/425-OLA-3 EXHIBIT NO. II-197
In the matter of Georgia Power Co. et al., Vogtle Units 1 & 2
 Staff Applicant Intervenor Other
 Identified Received Rejected Reporter SD
Date 9/14/95 Witness HILL and WARD

Following the SAE, the NRC has inspected ED&E activities and reviewed root cause/corrective action activities on a frequent basis. The most recent NRC inspection addressing Calcon sensors was conducted May 9-20, 1994, (Inspection Report 50-424, 425/94-12, dated June 9, 1994.) The inspectors did not identify any violations associated with these sensors. In addition, the inspectors identified that you had corrected the deficiencies that existed in the March 1990, time frame and few failures have been experienced since that time. The NRC inspections have confirmed that your Safety Audit and Engineering Review group has conducted adequate root cause analyses and GPC has corrected similar deficiencies.

Based on the reviews conducted in 1990 and subsequent observations, no further actions are necessary and this action is considered closed.

Should you have any questions concerning this letter, please contact us.

Sincerely,



Ellis W. Marschoff, Director,
Division of Reactor Projects

Docket Nos.: 50-424, 50-425
License Nos.: NPF-68, NPF-81

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(cc cont'd - See page 3)