NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
PALISADES NUCLEAR PLANT	0 5 0 0 0 2 5 5	1 OF 0 2

TITLE (4)
Containment Temperature

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									NA	0 5 0 0 0
1 0	2 2	8 4	8 4	— 0 2 2	— 0 0	1 1	2 1	8 4	NA	0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
David W. Rogers; Technical Engineer; Palisades	AREA CODE 6 1 6 7 6 4 - 8 9 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The Palisades Plant analysis of peak containment building pressure following a postulated Main Steam Line Break was discovered to contain an incorrect value for the initial containment air temperature. Temperatures inside the containment building routinely exceed the assumed temperature of 104 degrees F, resulting in higher calculated containment pressure. Initial containment temperature was not previously identified as a significant parameter relative to peak containment pressure. Subsequently, temperature limits have been established and implemented for various Plant operating conditions.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On September 26, 1984, with the Plant in cold shutdown condition, the containment building [NH] air temperature was discovered to be routinely greater than the 104 degrees F value assumed in the Main Steam Line Break accident analysis. The analysis determined that following a Main Steam Line Break (MSLB) with a loss of off-site power and failure of one diesel generator [DG;EK], peak containment pressure may exceed the containment design limit of 55 psig. Subsequent analysis determined that the 55 psig limit would not be exceeded unless the average initial temperature was in excess of 137 degrees F, rather than 104 degrees F.

Records indicate that on July 4, 1983, a temperature of 138 degrees F was measured in the containment dome. No other readings above 137 degrees F were recorded in 1983 or 1984 while the Plant was in operation. The occurrence was determined to be reportable on October 22, 1984.

The cause of the error has been attributed to personnel error. Although it remains unknown how the initial temperature value of 104 degrees F came to be used, it is evident that initial containment temperature was not previously identified as a significant variable and was, therefore, not monitored for purposes related to peak containment pressure.

Containment temperature limits have subsequently been established and implemented for various Plant operating conditions, which will ensure that peak containment pressure will remain below the design limit following a postulated MSLB.

Although a temperature of 138 degrees F was measured in the containment dome on July 4, 1983, three additional temperature indicators at various other locations inside containment indicated temperatures of 120, 122 and 130 degrees F. Therefore, the average containment temperature would presumably be less than 138 degrees F. Since an accurate method of averaging containment temperature readings has not been developed, the highest temperature was conservatively taken as the average containment temperature. Consequently, should the postulated MSLB have occurred on July 4, 1983, containment pressure would not have exceeded the design limit, and no threat to public health or safety would have resulted.

Consumers Power Company

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November 21, 1984

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-22 (CONTAINMENT TEMPERATURE)

Attached please find Licensee Event Report 84-22 (Containment Temperature) which is reportable to the NRC per 10 CFR 50.73(a)(2)(ii) and 10 CFR 50.73(a)(2)(v).

Brian D Johnson
Staff Licensing Engineer

CC Director, Office of Nuclear Reactor Regulation
Director, Office of Inspection and Enforcement
NRC Resident Inspector - Palisades

Attachment

OC1184-0006A-NL02

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