

APPENDIX

U.S. NUCLEAR REGULATORY COMMISSION  
REGION IV

NRC Inspection Report No. 50-482/92-19

Operating License No. NPF-42

Licensee: Wolf Creek Nuclear Operating Corporation  
P.O. Box 411  
Burlington, Kansas 66839

Facility Name: Wolf Creek Generating Station (WCGS)

Inspection At: Burlington, Kansas

Inspection Conducted: July 13-17, 1992

Inspector: R. C. Stewart, Reactor Inspector, Materials and Quality Programs  
Section, Division of Reactor Safety

Approved: *J. Barnes*  
I. Barnes, Chief, Materials and Quality  
Programs Section, Division of Reactor Safety

8-18-92  
Date

Inspection Summary

Inspection Conducted July 13-17, 1992 (Report 50-482/92-19)

Areas Inspected: Routine, announced inspection of the inservice inspection (ISI) results from the fifth refueling outage and followup of a previously identified inspection finding.

Results: There were no violations or deviations identified. In general, the WCGS ISI examinations performed during the fifth refueling outage appeared to meet the applicable ASME Code, Section XI requirements and the ISI program accepted by the NRC. The inspector also observed that the examination results were well organized in clearly identifiable data file volumes which contained the appropriate equipment calibration data, inspection sheets, and certification of inspection personnel. This observation was considered an apparent strength in the licensee's ISI program document control.

The following previously identified inspection finding was dispositioned as indicated:

- o Inspection Followup Item 482/9117-01 (CLOSED)

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DETAILS

1. PERSONS CONTACTED

WCNOC PERSONNEL

- \*R. Holloway, Manager, Maintenance and Modifications
- \*P. Clarkson, Supervisor, Mechanical Engineering
- \*M. Williams, Manager, Plant Support
- \*R. Flannigan, Manager, Nuclear Safety Engineering
- \*S. Wideman, Supervisor, Licensing
- \*D. Fehr, Manager, Operations Training
- \*R. Hammond, Health Physicist
- \*G. Pendergrass, Supervisor Engineer - Inservice Inspection
- \*J. Weeks, Manager, Operations
- \*T. Damashek, Quality Assurance (QA) Surveillance Supervisor
- \*K. Harvey, Quality Control Supervisor
- \*M. Reed, Health Physicist
- \*R. Logsdon, Manager, Chemistry
- \*S. Holman, Health Physicist
- \*R. Ryan, QA Auditor
- \*R. Taylor, Health Physicist
- \*T. Conley, Health Physicist
- \*G. McClelland, QA Specialist
- \*D. Mosebey, Supervisor, Operations
- \*J. Winkel, Authorized Nuclear Inservice Inspector (ANII)
- \*G. Seier, Results Engineering
- \*C. Fowler, Manager, Instrumentation & Controls

NRC

- \*L. Myers, Resident Inspector
- \*L. Ricketson, Senior Radiation Specialist, Division of Radiation Safety and Safeguards
- \*C. Yates, Reactor Engineer Intern
- \*G. Pick, Senior Resident Inspector

\*Denotes persons present at the July 17, 1992 exit interview.

The inspector also contacted other licensee personnel during the course of the inspection.

2. FOLLOWUP ON PREVIOUS INSPECTION FINDING (92701)

(CLOSED) INSPECTOR FOLLOWUP ITEM (482/9117-01): Assessment of the effects on heat exchanger testing of recent changes made in applicable design heat loads.

The inspector reviewed the licensee's engineering interoffice memorandum (NP-92-0713, dated April 14, 1992) in which engineering stated that,

"Calculation GL-02-W," Revision 01 was issued on May 24, 1990. This calculation revised the penetration room cooler heat loads for post-LOCA marginally. For SGL15A, the heat load was revised from 110,273 BTU/hr to 108,274 BTU/hr and for SGL15B this heat load was revised from 117,631 BTU/hr to 111,712 BTU/hr.

WCNOC-62 compiles design basis information and operating conditions for various safety-related components cooled by essential service water/normal service water. Since Generic Letter 89-13 required testing of penetration room coolers, the test was scheduled for RF-V outage (September 1991) and the heat loads had decreased marginally. Engineering did not consider the need to revise WCNOC-62 immediately after the issuance of GL-02-W, Revision 01, as no safety concern existed. WCNOC-62, Revision 01 was issued on July 17, 1991, prior to the test schedule.

The inspector determined that the licensee's position on this item was appropriate, and verified that the test results for the penetration room coolers were satisfactory with respect to WCNOC-62, Revision 01, requirements. This inspector followup item is considered closed.

### 3. INSERVICE INSPECTION (ISI) DATA REVIEW AND EVALUATION (73755)

The purpose of this inspection was to ascertain whether: (a) the data from the current ISI activities reflected the scope of examinations required during the current inspection period as described in Section XI of the American Society of Mechanical Engineering (ASME) Boiler and Pressure Vessel Code, the Technical Specifications, and the ISI program accepted by the NRC; (b) the ISI data files were complete and the data was within the previously established acceptance criteria; and (c) the licensee's disposition of adverse findings and subsequent re-examination were consistent with regulatory requirements.

The inspector conducted an overview of the WCNOC ISI program plan and found that ISI activities were performed by Nuclear Energy Services, with the exception of the centrifugally cast stainless steel weld volumetric examinations which were performed by Babcock & Wilcox Nuclear Services Company under separate contract. The ISI program plan was developed to cover the complete nondestructive examination requirements of all systems within the Class 1, 2, and 3 boundaries in accordance with ASME Section "I, 1980 Edition through the 1981 Winter Addendum, including repairs, replacements, modifications, and system hydrostatic testing.

The inspector reviewed the ISI summary report for the fifth refueling outage (RF-V) dated April 14, 1992, to verify that the scope of examinations was consistent with that required by the ISI program plan. In addition, the inspector selected eight components from the ISI summary report (see the Attachment to this report) and reviewed the examination results with respect to compliance with program and procedural requirements for: (a) method, extent, and technique of examination; (b) acceptance criteria; (c) recording, evaluating, and disposition of findings; (d) calibration of ultrasonic equipment; (e) identification of penetrant materials; and (f) use of

appropriately certified personnel for evaluation of examination data. Also reviewed during this inspection, were the results of the ASME Code, Section XI, reactor coolant system pressure tests and two modifications involving Code Class 1 and 2 (see the Attachment to this report).

The inspector noted from this review that the examinations appeared to be consistent with ASME Code Section XI, ISI program, and procedural requirements. The inspector also observed that the ISI data was well organized in clearly identified data file volumes, which contained the appropriate examination results, equipment calibration data, inspection sheets, and certification of inspection personnel. This observation was considered a strength in the licensee's ISI program document control.

4. EXIT INTERVIEW

The inspection scope and findings were summarized in an exit meeting on July 17, 1992, with the personnel listed in paragraph 1 of this report. The licensee did not identify as proprietary any of the materials provided to, or reviewed by, the inspector during this inspection.

ATTACHMENT

LIST OF DOCUMENTS REVIEWED

INSPECTION FOLLOWUP ITEM

- o Room Cooler Heat Transfer Verification and Performance Trending, Surveillance Test Results, Procedure STN PE-036, dated October 2, 1992, for Unit SGL15A and November 21, 1991, for Unit SGL15B

ISI PROGRAM DOCUMENTS

- o ISI Program Plan, WCRE-07
- o WCNOI NES Contract, M-189
- o NES ISI Program Plan, 83A1692, Revision 0
- o Inservice Inspection Report (Fifth Refueling Outage), dated April 14, 1992

ISI COMPONENT EXAMINATION DATA

<u>COMPONENT ID</u>	<u>TYPE EXAMINATION</u>
BB-01-F204	PT
EJ-03-FW204	PT
AB-01-046-M	UT
BB-01-F201	UT
BB-01-F204	UT
BB-01-S301-6	UT
BB-04-FW302	UT
EBB01A-2-A-(1-16)6	VT-1

REPAIR/MODIFICATION

<u>COMPONENT ID</u>	<u>WORK ORDER</u>
BG HV8153A BG HV8153B	91-061 dated March 17, 1992
EPV0097 EP-084-ECB-3/8"	91-179 dated December 21, 1991

ASME CODE - PRESSURE TEST

Reactor Coolant System Pressure Tests, Procedures PE-040A, and PE-040B, dated October 8, 1991, November 17, 1991, and January 9, 1992