



UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 REGION II
 101 MARIETTA STREET, N.W.
 ATLANTA, GEORGIA 30323

FEB 08 1985

Report Nos.: 50-348/85-01 and 50-364/85-01

Licensee: Alabama Power Company
 600 North 18th Street
 Birmingham, AL 35291

Docket Nos.: 50-348 and 50-364 License Nos.: NPF-2 and NPF-8

Facility Name: Farley 1 and 2

Inspection Conducted: January 7-11, 1985

Inspectors:	<u>P.G. Stoddart</u>	<u>2/6/85</u>
	P. G. Stoddart	Date Signed
	<u>P.G. Stoddart / for</u>	<u>2/6/85</u>
	J. D. Harris	Date Signed
	<u>P.G. Stoddart / for</u>	<u>2/6/85</u>
	W. B. Gloersen	Date Signed
Approved by:	<u>Douglas M. Collins</u>	<u>2/6/85</u>
	Daniel M. Montgomery, Section Chief	Date Signed
	Emergency Preparedness and Radiological	
	Protection Branch	
	Division of Radiation Safety and Safeguards	

SUMMARY

Scope: This routine, unannounced inspection entailed 116 inspector-hours on site in the areas of liquid and gaseous radioactive waste management, effluent monitoring, and environmental monitoring.

Results: No violations or deviations were identified.

REPORT DETAILS

1. Licensee Employees Contacted

- *J. D. Woodard, Plant Manager
- *W. G. Ware, Supervisor, Safety Audit and Engineering Review Group
- *M. Mitchell, Health Physics Supervisor
- *R. Bayne, Chemistry and Environmental Supervisor
- *D. E. Grissette, Environmental and Emergency Planning Supervisor
- *C. D. Nesbitt, Technical Superintendent
- *L. A. Ward, Maintenance Superintendent
- *W. B. Shipman, Assistant Plant Manager - Support
- *R. D. Hill, Operations Superintendent
- *D. N. Morey, Assistant Plant Manager - Operations
- J. M. Walden, Radwaste Supervisor
- D. Moore, Health Physicist
- K. Riley, Computer Services Technician
- L. Huey, Systems Engineer
- J. Kelly, Health Physicist
- R. Wood, Plant Chemist
- L. Drew, Health Physics Technician

NRC Resident Inspectors

- *W. H. Bradford

- *Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on January 11, 1985, with those persons indicated in paragraph 1 above.

The licensee was informed that the evaluation presented in response to IFI 348, 364/83-28-02 (Evaluate Particulate and Iodine Sample Losses in Long Runs if Sample Delivery Lines) would be reviewed by the regional staff. In a telephone conversation between the licensee's Technical Superintendent and the inspector on January 31, 1985, the licensee was informed that the portion of the evaluation concerning particulate losses was acceptable but that the portion concerning iodine aerosol losses was not acceptable. Licensee agreed to re-evaluate the remaining concerns; however, no completion date was specified.

3. Audits and Appraisals (80721, 84723, 84724)

Technical Specification 6.5.2.8 requires audits of the radiological environmental monitoring program at least once each 12 months, of the Offsite Dose Calculation Manual (ODCM) and implementing procedures at least once each 24 months, and of the Process Control Program (PCP) and implementing procedures

at least once each 24 months. Technical Specification 6.2.3 designates the Safety Audit and Engineering Group as being responsible for audit requirements.

The following audits and appraisals were performed during 1984 and were reviewed by the inspectors: Offsite Calculation Manual, March 7, 1984, Radioactive Waste Management Audit, May 29 - July 3, 1984; Health Physics and Radiological Controls Audit, October 5, 1984, Chemistry and Chemistry STPs Audit, October 30, 1984 and Process Control Program Audit, July 3, 1984. The inspector also reviewed the Environmental Monitoring Audit, dated March 12, 1984. Audits of the licensee's current radiological environmental sample analysis contract laboratory, the University of Georgia's Center for Applied Isotope Studies, were conducted on October 8, 1982 and October 6, 1983 by the Safety Audit and Engineering Review group located at the Licensee's corporate office. Neither report indicated any noncompliances. The contract laboratory was not audited in 1984; however, a joint audit with Georgia Power Company coordinated by Southern Company Services is scheduled for January 22, 1985.

No violations or deviations were identified.

4. Changes to Equipment and Procedures (80721, 84723, 84724)

Within the review areas of this inspection, no changes to liquid or gaseous effluent treatment or effluent monitoring systems or to the environmental monitoring program had been made since the previous inspection.

Technical Specification 6.8.1 requires the licensee to establish, implement, and maintain written procedures. The inspectors reviewed the following procedures. All procedures listed had been reviewed and approved by appropriate plant management, as provided in the Technical Specification.

FNP-0-CCP-202, Water Chemistry Specifications, Rev. 0, August 24, 1984

FNP-0-CCP-201, Schedule, Chemistry and Water Treatment Plant Activities, Rev. 0, December 7, 1984

FNP-1-CCP-304, Chemical Addition/Control of the Primary Coolant, Rev. 2, August 6, 1984

FNP-1-CCP-305, Chemical Addition/Control of the Secondary System, Rev. 3, October 26, 1984

FNP-0-RCP-274, Radiological Calibration of the Main Steam Relief and Atmospheric Steam Dump Discharge and Auxiliary Feedpump Turbine Exhaust, Rev. 4, July 9, 1984

FNP-0-RCP-276, Isotopic Calibration of Turbine Building Ventilation Exhaust Monitor, Rev. 3, March 6, 1984

- FNP-0-RCP-277, Isotopic Calibration of the Inline Westinghouse Liquid Radiation Monitors, Rev. 3, October 28, 1982
- FNP-0-ENV-1, Assignment of Classification Code Numbers to Environmental Radioactivity Samples, Rev. 6, October 21, 1983
- FNP-0-ENV-2, Sampling of Milk from Dairy Cows or Goats for Radioactivity Analysis, Rev. 10, October 31, 1983
- FNP-0-ENV-3, Air Sampling for Radioactive Particulates and Radioiodine Analysis, Rev. 5, May 8, 1984
- FNP-0-ENV-4, Measurement of Direct Gamma Radiation Using LiF Thermoluminescent Dosimeters, Rev. 7, August 30, 1984
- FNP-0-ENV-5, Sampling of Water in the Chattahoochee River for Radioactivity Analysis, Rev. 8, November 29, 1983
- FNP-0-ENV-7, Sampling of Ground Water for Radioactivity Analysis, Rev. 5, May 4, 1983
- FNP-0-ENV-8, Sampling of Vegetables and Fruits Grown in the Vicinity of the Farley Nuclear Plant for Radioactivity Analysis, Rev. 3, April 11, 1983
- FNP-0-ENV-9, Collection of Fish from the Chattahoochee River for Radiological Analysis, Rev. 5, February 24, 1984
- FNP-0-ENV-10, Sampling of Soil for Radioactivity Analysis, Rev. 0, November 8, 1984
- FNP-0-ENV-13, Environmental Data Review and Data Handling, Rev. 3, May 4, 1983
- FNP-0-ENV-15, In-situ Soil Monitoring, Rev. 4, December 8, 1984
- FNP-0-ENV-16, Sediment Collection, Rev. 5, November 4, 1983
- FNP-0-ENV-17, Meteorological Tower Support Activities, Rev. 5, May 7, 1984
- FNP-0-ENV-101, Schedule, Environmental Monitoring Program, Rev. 8, October 31, 1983

No violations or deviations were identified.

5. Semi-Annual and Annual Reports (80721, 84723, 84724)

Technical Specification 6.9.1.6 requires the licensee to submit an Annual Radiological Environmental Report. The inspectors reviewed the report for Calendar Year 1983. The report for Calendar Year 1984 was not available for review at the time of the inspection.

Technical Specification 6.9.1.8 requires the licensee to submit a Semi-Annual Radiological Effluent Release Report. The inspector reviewed reports for January-June 1983, July-December 1983, and January-June 1984. No technical discrepancies were noted and the reports were consistent with the guidance of Regulatory Guide 1.21.

On the basis of the Semi-Annual Effluent Release Reports discussed above, and on the basis of the inspector's review of liquid and gaseous effluent release permits, the inspector concluded that plant effluents were within Appendix I (10 CFR Part 50) design objectives, and were ALARA (As Low As Reasonably Achievable).

No violations or deviations were identified.

6. Radioactive Liquid Wastes and Liquid Effluent Treatment Systems (84723)

Technical Specifications 3.11.1.1, 3.11.1.2 and 3.11.1.3 establish limits for concentrations of radioactive materials in liquid effluents, require the liquid effluent treatment system to be operable, and require the use of the liquid effluent treatment system under certain conditions.

The inspectors reviewed selected liquid effluent release permits for the period July 1, 1984 through December 31, 1984.

Technical Specification 3.11.1.3 requires the liquid radwaste system to be operable. The Technical Specification further requires the appropriate portions of the liquid radwaste system to be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to liquid effluents from the site, when averaged over the calendar quarter, would exceed 0.18 mrem to the total body or 0.6 mrem to any organ.

The inspectors reviewed selected operating records for the liquid radwaste system and determined that calculated doses had not exceeded the specified limits during the period July 1, 1984 to December 31, 1984; however, the licensee had elected to use the liquid radwaste system for the entire period to minimize the radioactivity content of effluents.

Technical Specification 3.11.1.2 requires the licensee to limit the dose or dose commitment to an individual from radioactive materials in liquid effluent releases to:

- a. During any calendar quarter to less than or equal to 1.5 mrem to the total body and to less than or equal to 5 mrem to any organ, and
- b. During any calendar year to less than or equal to 3 mrem to the total body and to less than or equal to 10 mrem to any organ.

The inspectors reviewed the ODCM and corresponding implementing procedures, as well as selected release permits and dose calculation records to meet the requirements of Technical Specification 3.11.1.2.

The inspectors verified from selected records of liquid effluent releases made during the period of July 1, 1984 through December 31, 1984, that the records required by Technical Specification 6.10 were maintained in terms of frequency and content.

The Farley Nuclear Plant liquid radwaste treatment system utilized disposable charcoal filter beds and disposable mixed-bed demineralizers for treatment of radioactive liquid wastes. Standard procedure was to use a three cell series arrangement consisting of a charcoal filter unit and two mixed-bed demineralizers. The charcoal filter unit and the demineralizer units each had a 170 ft³ internal volume. Input to the liquid radwaste system over the past 18 months averaged between 3500 gpd and 4,000 gpd per unit.

Farley has had a formal leak reduction program in place for several years. The program included a tracking system and a daily water report distributed to principal staff. Graphic trending was also performed regularly using computer charting programs. Licensee personnel credited the plant's success in maintaining low waste water input and inventory to the various tracking systems and to active management support of the program.

The 3-cell waste water treatment method had been in use for over four years. In that time, only eight resin beds were changed-out and through-put was approximately 10⁶ gallons per resin bed expended. Costs were estimated at around \$0.085 per gallon compared to earlier plant experience of \$0.44 per gallon for evaporative treatment.

Expanded resins were dewatered in-place in the disposable liner and were shipped offsite for disposal.

No violations or deviations were identified.

7. Radioactive Gaseous Waste and Gaseous Effluent Treatment Systems (84724)

Technical Specification 3.11.2.1 requires that the dose rate due to radioactive materials released in gaseous effluents from the site shall be limited to:

- a. For noble gases: less than or equal to 500 mrem/yr to the total body and less than or equal to 3000 mrem/yr to the skin, and
- b. For radioiodines and for all radioactive materials in particulate form and radionuclides (other than noble gases) with half lives greater than 8 days: less than or equal to 1500 mrem/yr to any organ.

Technical Specification 3.11.2.2 requires that the air dose due to noble gases released in gaseous effluents, from each reactor unit, to areas at or beyond the site boundary shall be limited to:

- a. During any calendar quarter: Less than or equal to 5 mrad for gamma radiation and less than or equal to 10 mrad for beta radiation and,

5. During any calendar year: Less than or equal to 10 mrad for gamma radiation and less than or equal to 20 mrad for beta radiation.

Technical Specification 3.11.2.3 requires that the dose to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each reactor unit to areas at and beyond the site boundary shall be limited to:

- a. During any calendar quarter: Less than or equal to 7.5 mrems to any organ and,
- b. During any calendar year: Less than or equal to 15 mrems to any organ.

Technical Specifications 3.7.7, 3.7.8, and 4.9.14 require that the ventilation exhaust treatment systems and the gaseous radwaste treatment system shall be operable.

Technical Specification 3.11.2.6 requires that the quantity of radioactivity contained in each gas storage tank shall be limited to less than or equal to 70,500 curies of noble gases (considered as Xe-133).

The inspectors reviewed selected records and logs of gaseous effluent releases for the period of July 1, 1984 through December 31, 1984.

The inspectors also reviewed the ODCM and relevant implementing procedures for determining projected offsite doses resulting from gaseous effluent releases.

The inspectors verified from selected records of gaseous effluent releases made during the period from July 1, 1984 to December 31, 1984, that the records required by Technical Specification 6.10 were maintained in terms of frequency and content.

No violations or deviations were identified.

8. Testing of Air Cleaning Filtration Systems (84724)

Technical Specifications 3.7.7, 3.7.8, and 3.9.14 provide requirements for the testing of charcoal adsorber sample retention efficiency for methyl iodide and for in-place leak testing of HEPA filtration and charcoal adsorption sections of exhaust and atmosphere cleanup filtration systems.

Filtration systems at Farley include the control room emergency air cleanup systems, the penetration room filtration systems and the containment purge exhaust filter. The inspectors reviewed selected records and logs of the operability of filtration systems. The inspectors reviewed test records for vendor tests of methyl iodide retention efficiency of adsorber charcoal samples. The inspector also reviewed test records of vendor leak tests of filtration systems.

No violations or deviations were identified.

9. Post-Accident Sampling System (84723)

Technical Specification 6.8.3.d requires the licensee to have the installed capability of sampling and analyzing plant fluids and gases in the event of an accident. The inspector reviewed procedures and records for the operation, maintenance, and testing of the licensee's Post-Accident Sampling System (PASS).

The inspectors, accompanied by a licensee representative, toured the PASS facility and discussed system operation and operability with facility operators.

In a letter dated January 4, 1985, the licensee responded to several unresolved issues identified in the evaluation of the PASS systems for Units 1 and 2 which was conducted by a NRC contractor during the week of June 25, 1984, and reported in 50-348, 364/84-18. The inspector reviewed the letter and discussed the response with licensee representatives. The inspector determined that most of the items of concern had been resolved.

Unresolved Item 50-348, 364/84-18-02 concerned the licensee's agreement to evaluate the capability of the volume control tank (VCT) to accommodate the volume of liquid waste expected to be generated from sampling during an accident and to evaluate the long-term consequences of using the VCT for liquid waste collection. The licensee's evaluation showed that the VCT could be used for up to two years of PASS operation under certain conditions and for up to 21 weeks under more conservative conditions. When necessary, the VCT contents could be returned to the Reactor Coolant System without violating the NUREG-0737 criteria against the use of auxiliary systems. The inspector found the licensee's response acceptable.

- (Closed) URI (Unresolved Item) 348, 364/84-18-02, Licensee to evaluate capacity of volume control tank (VCT) and long-term consequences of returning liquid waste to the VCR from the PASS.

Inspector Followup Item (IFI 50-348, 364/84-18-03) was applicable to both Units 1 and 2 and had eight subparts.

Subpart 1 concerned the licensee's agreement to evaluate modifications to installed systems and procedures with respect to the original shielding evaluation. The licensee's response stated that an evaluation indicated no increase in expected exposures resulting from the modifications.

Subpart 2 concerned auxiliary ventilation of the Unit 1 PASS sampling station. Licensee stated that modifications had been scheduled to be completed approximately April 1985. In the interim, the auxiliary ventilation system would remain in use. This item was re-listed separately:

Inspector Followup Item (IFI 50-348/85-01-01: Modify Unit 1 PASS Ventilation System to Prevent Outleakage of Air.)

Subpart 3 concerned provisions for purging sample lines. The Licensee evaluation maintains that the intent of NUREG-0737 criterion (11) for PASS systems was sample validity and that "purging" should be interpreted as providing a representative sample and not a clearing or flushing of lines to reduce potential personnel exposures. The inspector found the Licensee's interpretation to be acceptable.

Subpart 4 concerned the recommended accuracy for analysis of dissolved hydrogen. A statement was made in the inspection report that an accuracy of $\pm 5\%$ for samples less than 50 cc/kg can be acceptable. This was a typographical error and should have read "... of ± 5 cc/kg for samples less than 50 cc/kg can be acceptable". The licensee stated that extensive testing indicated that an accuracy of $\pm 20\%$ could be attained with the Unit 2 system but that modifications to the Unit 1 system were necessary to achieve the same degree of accuracy. The licensee stated that they considered $\pm 20\%$ to be a reasonable value at the subject concentrations. It was pointed out that ± 5 cc/kg at 50 cc/kg was $\pm 10\%$ but that at 25 cc/kg, ± 5 cc/kg was $\pm 20\%$. The inspector discussed the licensee's position with NRR Chemical Engineering personnel with the consensus that the licensee's position for $\pm 20\%$ at 50 cc/kg was acceptable. Licensee committed to modify the Unit 1 PASS system to improve dissolved hydrogen analysis capability by approximately September 1987. This item was re-listed as a separate IFI:

Inspector Followup Item (IFI 50-348/85-01-02, Modify Unit 1 PASS to Improve Hydrogen Sampling Capability).

Subpart 5 was concerned with training and requalification for PASS operators. The licensee had revised the training program and included semi-annual hands-on training for concerned personnel.

Subpart 6 was concerned with the absence of a periodic maintenance and testing program for the PASS. The licensee had placed applicable equipment on the station preventive maintenance program.

Subpart 7 was concerned with the potential for spillage of primary coolant sample material during the filling operation. The licensee evaluated the problem and concluded it could be resolved by training and practice, which has been included in the training program noted under subpart 5 above.

Subpart 8 was concerned with the licensee's analytical methods and the capability of analyzing a standard test matrix to the desired degree of accuracy. The licensee performed an analysis of a test matrix in accordance with an NRR requirement. The results were within the specified level of accuracy.

(Closed) IFI 348, 364/84-18-03, Licensee to evaluate eight concerns in regard to the PASS.

No violations or deviations were identified.

10. Effluent Radiological Monitoring Instrumentation (84723, 84724)

Technical Specification 3.3.3.10 requires that liquid effluent radioactivity monitors shown in Technical Specification table 3.3.13 shall be operable with their alarm/trip setpoints set to ensure that the limits of Technical Specification 3.11.1.1 are not exceeded.

The inspectors reviewed appropriate sections of the licensee's ODCM and corresponding implementing procedures for determining and adjusting alarm/trip setpoints. The inspectors reviewed selected instrument maintenance logs and records for instrumentation setpoints and verified that the bases for the setpoints were understood by affected operators and technicians.

Technical Specification Table 4.3.8 provides surveillance requirements for liquid effluent radioactivity monitors.

The inspectors reviewed selected logs and records and verified that the required channel calibration and channel functional surveillance tests had been performed at the prescribed intervals.

Table Notation (2) of Technical Specification Table 4.3.8 provides requirements for channel calibration to appropriate reference standards obtained from suppliers that participate in measurement assurance activities with the National Bureau of Standards.

The inspectors reviewed selected logs and records and verified that channel calibration had been performed in accordance with the specification.

Technical Specification 3.3.3.11 requires that gaseous effluent radioactivity monitors shown in Technical Specification Table 3.3.14 shall be operable with their alarm/trip setpoints set to ensure that the limits of Technical Specification 3.11.2.1 are not exceeded.

The inspectors reviewed appropriate sections of the licensee's ODCM and corresponding implementing procedures for determining and adjusting alarm/trip setpoints. The inspectors also reviewed selected instrument maintenance logs and records for instrumentation setpoints and verified that the bases for the setpoints were understood by affected operators and technicians.

Technical Specification Table 4.3.9 provides surveillance requirements for gaseous effluent radioactivity monitors.

The inspectors reviewed selected logs and records and verified that the required channel calibration and channel functional surveillance tests had been performed at the prescribed intervals.

Table Notations (2), (3), and (4) of Technical Specification Table 4.3.9 provide requirements for channel calibration to appropriate reference standards.

The inspector reviewed selected logs and records and verified that channel calibration had been performed in accordance with the Technical Specification.

The inspectors toured the effluent radioactivity monitoring stations. The inspectors observed that no corrective action had been taken to resolve a previous inspection item (IFI 50-348, 364/83-28-01) by replacing or correcting several sharp right angle bends in the sampling lines leading to the high range gaseous effluent radioactivity monitors. The inspectors reviewed proposed Product Change Requests (PCRs) to accomplish necessary modifications. The inspectors notified licensee representatives that the item would remain open until the work had been completed.

No violations or deviations were identified.

11. Reactor Coolant Chemistry (84723)

Technical Specification Table 4.4-4 specifies sampling and analysis frequencies for coolant analyses for gross radioactivity, dose equivalent I-131, radiochemical E determination, and isotopic analyses for I-131, I-133, and I-135.

Technical Specification Table 3.4-2 specifies the maximum coolant concentration limits for dissolved oxygen, chloride, and fluoride when the coolant temperature is above 250°F. Sampling frequencies are specified in Technical Specification Table 4.4-3. The inspectors reviewed selected plant chemistry records for the period July-13, 1984 through December 31, 1984 and verified that the required tests were performed at the specified frequencies and that the results were within specified limits. Technical Specification 3.7.1.4 requires a program for monitoring specific activity in the secondary coolant system. The inspectors reviewed the licensee's procedure for monitoring specific activity in the secondary coolant and examined selected logs and records of the program.

No violations or deviations were identified.

12. Radiological Environmental Monitoring (80721)

The inspectors reviewed the licensee's environmental monitoring program as required by Technical Specification 3.12.1. The licensee uses the University of Georgia's Center for Applied Isotope Studies (CAIS) for analyzing radiological environmental samples. The contract laboratory participates in EPA's Environmental Radioactivity Laboratory Intercomparison Study. The inspectors reviewed the results of the various gamma emitters in water and tritium intercomparison data published in February 1984. All but one of the CAIS results were within the EPA's established acceptance criteria.

The inspectors reviewed the 1983 Annual Environmental Operating Report issued on April 30, 1984. The 1984 Annual Environmental Operating Report was in preparation at the time of this inspection. The inspectors noted in the 1983 report that several quarterly and annual TLDs were reported as

missing. A licensee representative informed the inspector that several TLDs were missing from various locations in 1984 and that these affected TLD stations were moved to safer locations (e.g., within the fenced area of a substation co-located with the affected TLD station). The inspector also noted that the licensee's contract carrier had lost five particulate air samples in 1983. The licensee took corrective action and used another contract carrier. The inspectors reviewed selective environmental monitoring sample collection records and analysis results for the period between January 1984 and December 1984 and verified that the required samples were collected at the specified frequencies described in the technical specifications.

The inspectors, accompanied by a licensee representative, inspected nine offsite environmental monitoring stations, verified the operation of seven air sampling stations, verified the presence of TLD packets at each location, and observed the collection of two river water samples. The inspectors noted that the licensee installed new air monitoring equipment at all locations in August/September 1984. The new equipment included totalizers, flow meters, timers, vacuum pumps, and housings. The inspector reviewed appropriate calibration records and determined that air sampling totalizers in use had been calibrated. The calibration frequency for the totalizers is once per six months.

The inspectors verified by direct observation and by record review that the required primary and backup meteorological monitoring stations' sensors and local readouts were operable and maintained. The inspectors observed, however, that the ambient temperature sensors on the primary tower had been out of service for over two years. The licensee used surveillance test procedures FNP-0-STP-752 (12/14/84, Rev. 3) and FNP-0-STP-255.1 (9/25/84, Rev. 0) for calibrating the primary and backup towers, respectively. The calibration records of all meteorological instruments were reviewed and it was determined that they were calibrated semi-annually in 1984. The inspectors observed that the meteorological readouts in the control room were not functioning properly. The chart drive motor operating the strip chart recorder in the control room had been out of service since 12/18/84. A work order was initiated on that same date and the licensee was waiting for the vendor to ship the spare part. Currently, the licensee is able to obtain time-averaged meteorological data in the technical support center which is adjacent to the control room.

No violations or deviations were identified.

13. Inspector Followup (92701)

The following inspector followup items were reviewed:

(Closed - Units 1 and 2) IFI 50-348, 364/84-13-02, IE Notice 84-22, Deficiency in COMSIP Standard Bed Catalyst. The licensee committed to replace the catalytic units.

(Closed - Units 1 and 2) IFI 50-348, 364/84-13-01, Corrective Action on R-18 Radwaste Process Monitor. Licensee took necessary corrective actions to decontaminate the sample chamber at more frequent intervals to minimize or prevent alarm conditions when the monitor is operated in the standby mode.

(Open - Units 1 and 2) IFI 50-348, 364/83-28-01, Reduce Particulate Sampling Losses by Eliminating Sharp Right Angle Bends in Sample Delivery Lines. Licensee had initiated a work request but the task had not been started at the time of the inspection.

(Closed - Units 1 and 2) IFI 50-348, 364/83-28-02, Evaluate Particulate and Iodine Sample Losses in Long Runs of Sample Delivery Lines. A licensee consultant's evaluation was reviewed by the regional staff. The portion of the evaluation concerning particulate aerosol losses was addressed adequately; however, the portion of the evaluation concerning iodine aerosol losses was not acceptable. The licensee was informed of the staff's determination on January 31, 1985, in a telephone conversation between the licensee's Technical Superintendent and the inspector. The licensee agreed to re-evaluate sampling line losses of iodine aerosols. A new IFI was opened to track the licensee's commitment:

Inspector Followup Item (50-348, 364/85-01-03): Evaluate iodine aerosol sample losses in long runs of sample delivery lines.

No violations or deviations were identified.