



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

August 17, 1992

Docket No. 50-333

Mr. Ralph E. Beedle
Executive Vice President, Nuclear
Generation
Power Authority of the State of
New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

SUBJECT: EVALUATION OF INTERGRANULAR STRESS CORROSION CRACKING (IGSCC)
INSPECTIONS CONDUCTED DURING THE 1992 REFUELING OUTAGE - JAMES A.
FITZPATRICK NUCLEAR POWER PLANT (TAC NO. M84114)

By letter dated July 17, 1992 (JPN-92-040), you submitted the results of the IGSCC inspections performed during the 1992 refueling outage to the NRC for review prior to startup. In that letter, you indicated that 43 welds were inspected by personnel certified in accordance with the Electric Power Research Institute - Boiling Water Reactor Owners Group IGSCC qualification program. The weld inspections included Categories A, C, D, and E welds as defined in Generic Letter (GL) 88-01 and NUREG-0313, Revision 2. As a result of the weld inspections, no IGSCC was discovered.

During the 1992 refueling outage, the "B" Core Spray piping, which included nine Categories D and E welds, was replaced. The replacement pipe and safe end are 347 modified stainless steel which is resistant to sensitization and classified as Category A. The piping design reduces the total number of welds in the piping run, including the safe end, to four welds.

During the 1992 refueling outage, your staff surface finished and inspected the overlay for weld 28-53. This standard weld overlay was installed during the 1991 FitzPatrick maintenance outage; however, by letter dated March 7, 1991, the NRC granted relief from surface finishing and ultrasonically inspecting the overlay at that time due to anticipated high personnel radiation exposure. Your staff surface finished and inspected weld 28-53 after the 1992 decontamination of the Reactor Recirculation system. The weld was found to be within all design requirements after surface finishing.

By letter dated November 13, 1991, the NRC staff requested that you report and evaluate the overlay shrinkage in weld 28-53. The NRC staff also requested that you assess the cumulative shrinkage effects resulting from weld overlay repairs and the effect of the increase in deadweight and stiffness on the piping systems and their supports and pipe whip restraints. The results of the assessment are scheduled to be submitted for NRC review no later than 90 days after restart from the 1992 refueling outage.

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Mr. Ralph E. Beedle

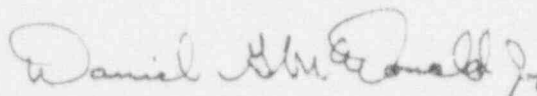
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August 17, 1992

Based on our review of the information provided, we find that the inspections that were performed meet the guidelines of Generic Letter 88-01. Therefore, we conclude that the FitzPatrick facility can be safely returned to operation for at least one additional operating cycle with assurance that the integrity of the reactor coolant pressure boundary will be maintained.

This concludes our review activities for TAC No. M84114.

Sincerely,



for Brian C. McCabe, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/11
Office of Nuclear Reactor Regulation

cc: See next page

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Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Based on our review of the information provided, we find that the inspections that were performed meet the guidelines of Generic Letter 98-01. Therefore, we conclude that the FitzPatrick facility can be safely returned to operation for at least one additional operating cycle with assurance that the integrity of the reactor coolant pressure boundary will be maintained.

This concludes our review activities for TAC No. M84114.

Sincerely,

Original Signed By: Daniel G. McDonald For

Brian C. McCabe, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

cc: See next page

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DATE	8/4/92	8/4/92	8/13/92	8/17/92	

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