

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 4 9	PAGE (3) 1 OF 0 2
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TITLE (4)  
Unit 3 Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	0	2	8	4		0	1	1	N/A		0 5 0 0 0
1	0	2	8	4		0	1	1	N/A		0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 3 4	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input checked="" type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 80.38(a)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 80.38(a)(2)	<input type="checkbox"/> 80.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 80.73(a)(2)(i)	<input type="checkbox"/> 80.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 80.73(a)(2)(ii)	<input type="checkbox"/> 80.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Daniel C. Kim	(Ext. 549)	AREA CODE 8 1 5	9 4 2 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	J	I	E C B D G	0 8 4	N					

SUPPLEMENTAL REPORT EXPECTED (14) <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)
		MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-spaced typewritten lines) (16)

During a planned unit shutdown due to steam leak in shaft seal system, reactor scrambled on MSIV not full open caused by Group I isolation on less than 850 psig with mode switch in run when turbine was manually tripped. The bypass valves opening all at once due to a faulty voltage comparator and lower than normal reactor pressure due to computer indication reading 10 psig too high, caused a Group I isolation. The damaged bellows were replaced in shaft seal system and the faulty voltage comparator was replaced as well as the computer points recalibrated. No further problems were noted after startup. Similar occurrence on the shaft seal system were reported in reportable occurrence 84-15 on Docket No. 050-249 and DVR 12-3-84-44.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	0 2 0	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Due to steam leak in turbine pipeway, Unit 3 was being shutdown on 10/26/84 to repair the shaft seal system. At 0908 hours and 34 percent power, reactor scrambled shortly after manual turbine trip. Reactor scram was due to MSIV not full open caused by the Group I isolation on less than 850 psig with mode switch in run.

The steam leak in turbine pipeway was due to damaged bellows on the seal steam relief valve discharge line. This condition was due to constant lifting and closing of seal steam header relief valve. Constant cycling of relief valve was probably due to seal steam bypass feed valve being failed in open position even though it indicated closed. Cause of seal steam bypass feed valve being failed in open position was due to improper limit switch setting. Also found seal steam feed regulating valve with the lower than designed air pressure setting. Bellows on discharge line were replaced. Also seal steam feed bypass valve, seal steam feed valve and seal steam feed relief valve were repaired and an adjustment for the air pressure setting for the seal steam feed regulating valve was performed.

The lower than normal reactor pressure was due to computer point on turbine throttle pressure indication being 10 psig too high. Consequently EHC pressure regulator was controlling turbine throttle pressure at 910 psig instead of 920 psig. Also low reactor pressure was due to failure of voltage comparator in EHC logic which caused all bypass valves to open at once on turbine trip instead of the pressure control unit sequentially opening the bypass valves. The bypass valves opening all at once coupled with lower than normal turbine throttle pressure caused reactor pressure to fall below the Group I isolation setpoint. All problems associated with shaft seal system, EHC system, and computer point indication were corrected. The unit started up after the repair and no further problems were noted. Similar occurrence on the shaft seal systems were reported in reportable occurrence 84-15, Docket No. 050-249 and DVR 12-3-84-44.



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November 19, 1984

DJS Ltr #84-1303

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Licensee Event Report #84-020-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(iv).

*Joh W. Wager*

*AS*

D.J. Scott  
Station Superintendent  
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III  
File/NRC  
File/Numerical

*IE22*  
*1/1*