

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3		DOCKET NUMBER (2) 0 5 0 0 0 3 6 2	PAGE (3) 1 OF 0 6
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TITLE (4)
DOSE EQUIVALENT IODINE LIMITS EXCEEDED

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																					
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																			
1	0	2 8	8 4	0 3 9	0 0 1	1 1	1 4	8 1 4			0 5 0 0 0 0 0 0																																			
<table border="1"> <tr> <td>OPERATING MODE (9) 3</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 0 0 0</td> <td>20.402(b)</td> <td>20.405(c)</td> <td>50.73(a)(2)(iv)</td> <td>73.71(b)</td> </tr> <tr> <td>20.405(a)(1)(i)</td> <td>50.36(c)(1)</td> <td>50.73(a)(2)(v)</td> <td>73.71(c)</td> </tr> <tr> <td>20.405(a)(1)(ii)</td> <td>50.36(c)(2)</td> <td>50.73(a)(2)(vii)</td> <td rowspan="3">X OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td>20.405(a)(1)(iii)</td> <td>50.73(a)(2)(i)</td> <td>50.73(a)(2)(viii)(A)</td> </tr> <tr> <td>20.405(a)(1)(iv)</td> <td>50.73(a)(2)(ii)</td> <td>50.73(a)(2)(viii)(B)</td> </tr> <tr> <td>20.405(a)(1)(v)</td> <td>50.73(a)(2)(iii)</td> <td>50.73(a)(2)(x)</td> <td></td> </tr> </table>												OPERATING MODE (9) 3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											POWER LEVEL (10) 0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	
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LICENSEE CONTACT FOR THIS LER (12)

NAME J. G. HAYNES, STATION MANAGER	TELEPHONE NUMBER 7 1 4 4 9 2 1- 7 1 7 0 1 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Coolant System specific activity.

On October 28, 1984, at 0225 with Unit 3 in Mode 3, following reactor shutdown for a steam generator repair outage, analysis of a Reactor Coolant System (RCS) sample indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by purification flow at 2030 on October 29, 1984.

The event was an indication of iodine spiking. We will continue to monitor and evaluate primary coolant activity. No further corrective action is planned.

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S PDR

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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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		YEAR	SEQ. NUMBER	REV. NUMBER		
		8 4	- 0 3 9	- 0 0	0 2	OF 0 6

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Coolant System (RCS) specific activity.

On October 28, 1984, at 0225 with Unit 3 in Mode 3, following reactor shutdown for Steam Generator (EIS Component Function Identifier SG) E-089 repair outage, RCS sample analysis indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by purification flow at 2030 on October 29, 1984.

The event was an indication of iodine spiking. Similar occurrences were previously reported in LER 83-111, LER 84-005, LER 84-013, LER 84-015, LER 84-023, LER 84-037 and LER 84-038. We will continue to monitor and evaluate primary coolant activity. No further corrective action is planned. Neither the health and safety of plant personnel nor the public were affected by this event.

Additional information, required by LCO 3.4.7, Action Statement 'd', is provided on the following pages. Although the unit has a degasification path which operates continuously and takes pressurizer steam, condenses it and directs it to Liquid Radwaste, degassing operation history is not applicable, because this system reduces the noble gas content of the RCS but has no effect on iodine.

LICENSEE EVENT REPORT (LER)
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

CLEANUP FLOW HISTORY

<u>PERIOD</u>	<u>AVERAGE CLEANUP FLOW (GPM)</u>
10/26/84, 0200 to 10/26/84, 0400	80.0
10/26/84, 0400 to 10/26/84, 0800	85.0*
10/26/84, 0800 to 10/28/84, 2300	75.3
10/28/84, 2300 to 10/29/84, 0100	85.0*
10/29/84, 0100 to 10/29/84, 2100	85.2

* hourly cleanup flow data not available. Figure used is taken from average flow with two charging pumps in operation.

REACTOR POWER HISTORY

<u>PERIOD</u>	<u>REACTOR POWER</u>
10/26/84, 0200 to 10/27/84, 1700	100% Rated Power
10/27/84, 1700 to 10/27/84, 1900	100% to 77%
10/27/84, 1900 to 10/27/84, 2200	77% to 32%
10/27/84, 2200 to 10/27/84, 2300	32% to 0%
10/27/84, 2300 to 10/29/84, 2100	0%

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY ANALYSIS

<u>DATE</u>	<u>TIME</u>	<u>DE I-131 MICROCURIES/GRAM</u>
10/28/84	0225	6.42
10/28/84	0430	6.15
10/28/84	0830	7.43
10/28/84	1230	6.21
10/28/84	1635	5.15
10/28/84	2035	3.81
10/29/84	0030	3.05
10/29/84	0430	2.66
10/29/84	0830	2.04
10/29/84	1230	1.54
10/29/84	1630	1.08
10/29/84	2030	0.78

The total time with the DE I-131 above 1.0 microcuries/gram for this event was 42.08 hours.

UNIT 3
SAN ONOFRE NUCLEAR GENERATING STATION
10/11/80

REPAIR NUMBER 12

FORM NO. 100
10/11/80

APPROVED (IND NO) 1100 01-84
REVISED (11-80)

CLASS IN

0 6 1 0 0 0 3 6 2 8 4 - 0 1 0 0 1 6 0 1 6

34-08 71-571 71-572 71-573 71-574 71-575 71-576 71-577 71-578 71-579 71-580 71-581 71-582 71-583 71-584 71-585 71-586 71-587 71-588 71-589 71-590

MINIMUM
PEAR
CARE AVERAGE

ENTRANCE ASSEMBLY POSITIVE 0-894690-BA-BAZT IN ASSEMBLY 10
PERMANENT FUEL ELEMENT INSURE 0-811701-003 NOV/77 OCCURRED AT 36-00 0/0 OF IM. CARE WEIGHT IN ASSEMBLY 108
Equal to 193.08 EFPD

BATCH NUMBER BATCH AVERAGE EXPOSURES AVERAGE POSITIVE (END/T)

71-571 71-572 71-573 71-574 71-575 71-576 71-577 71-578 71-579 71-580 71-581 71-582 71-583 71-584 71-585 71-586 71-587 71-588 71-589 71-590

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128
SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

November 14, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 84-039
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Coolant System specific activity. Neither the health and safety of plant personnel nor the public were affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 84-039

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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