



Entergy
Operations

Entergy Operations, Inc.

Route 2 Box 1373

Highland, AR 72061

Tel 501 954-3100

August 14, 1992

1CAN089203

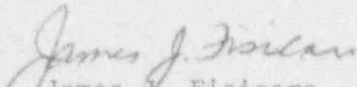
U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Stop P1-137
Washington, D.C. 20555

SUBJECT: Arkansas Nuclear One - Unit 1
Locket No. 50-313
License No. DPR-51
Monthly Operating Report

Gentlemen:

Monthly Operating Report statistics for Arkansas Nuclear One, Unit 1, for July, 1992 is attached. This report is submitted in accordance with ANO-1 Technical Specification 6.12.2.3.

Very truly yours,


James J. Fisicaro
Director, Licensing

JJF/SAB/sjf
Attachment

9208200236 920731
PDR ADDC 05000313
R PDR

IKDA
11

cc:

Mr. James L. Milhoan
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector
Arkansas Nuclear One - ANO-1 & 2
Number 1, Nuclear Plant Road
Russellville, AR 72801

Mr. Thomas W. Alexion
NRR Project Manager, Region IV/ANO-1
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, Maryland 20852

Ms. Sheri Peterson
NRR Project Manager, Region IV/ANO-2
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, Maryland 20852

OPERATING DATA REPORT

DOCKET NO: 50-313
 DATE: August 5, 1992
 COMPLETED BY: K. R. Hayes
 TELEPHONE: (501) 964-5535

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 1
2. Reporting Period: July 1-31, 1992
3. Licensed Thermal Power (MWt): 2,568
4. Nameplate Rating (Gross MWe): 902.74
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: None

	<u>MONTH</u>	<u>YR-TO-DATE</u>	<u>CUMULATIVE</u>
11. Hours in Reporting Period	744.0	5,111.0	154,434.0
12. Number of Hours Reactor was Critical	744.0	3,464.8	109,326.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	5,044.0
14. Hours Generator On-Line	744.0	3,415.9	107,148.7
15. Unit Reserve Shutdown Hours ..	0.0	0.0	817.5
16. Gross Thermal Energy Generated (MWH)	1,895,663.0	8,547,831.0	243,729,269.0
17. Gross Electrical Energy Generated (MWH)	633,200.0	2,901,800.0	81,180,340.0
18. Net Electrical Energy Generated (MWH)	605,266.0	2,763,933.0	77,139,181.0
19. Unit Service Factor	100.0	66.8	69.4
20. Unit Availability Factor	100.0	66.8	69.9
21. Unit Capacity Factor (Using MDC Net)	97.3	64.7	59.7
22. Unit Capacity Factor (Using DEC Net)	95.7	63.6	58.8
23. Unit Forced Outage Rate	0.0	0.1	12.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			

25. If Shut Down At End of Report Period. Estimated Date of Startup: _____
26. Units in Test Status (Prior to Commercial Operation): _____

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY	_____	<u>08/06/74</u>
INITIAL ELECTRICITY	_____	<u>08/17/74</u>
COMMERCIAL OPERATION	_____	<u>12/19/74</u>

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-313
 UNIT: One
 DATE: August 5, 1992
 COMPLETED BY: K. R. Hayes
 TELEPHONE: (501) 964-5535

MONTH July, 1992

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	828
2	827
3	826
4	823
5	823
6	822
7	820
8	820
9	819
10	817
11	818
12	818
13	818
14	819
15	820
16	821
17	822
18	822
19	820
20	820
21	820
22	821
23	821
24	810
25	622
26	815
27	817
28	817
29	817
30	818
31	820

AVGS: 814

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

MONTHLY OPERATING REPORT

OPERATING SUMMARY

JULY, 1992

UNIT ONE

Unit One began the month operating at 100% power. On the tenth at 19:40 hours, the unit load was decreased to 98% to perform scheduled testing of the turbine throttle/governor valves. The unit returned to 100% power at 21:25 hours on the same day. On the twenty fourth at 22:00 hours, the Load Dispatcher requested a power reduction. During the power reduction, a condenser tube leak was located and plugged, and the turbine throttle/governor valves were tested. The unit returned to 100% power on the twenty sixth at 01:20 hours. The unit operated at 100% power for the remainder of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT FOR JULY, 1992

DOCKET NO.	<u>50-313</u>
UNIT NAME	<u>ANO Unit 1</u>
DATE	<u>August 7, 1992</u>
COMPLETED BY	<u>K. R. Hayes</u>
TELEPHONE	<u>(501) 964-5535</u>

<u>No.</u>	<u>Date</u>	<u>Type</u>	<u>Duration (Hours)</u>	<u>Reason²</u>	<u>Method of Shutting Down Reactor³</u>	<u>Licensee Event Report #</u>	<u>System Code⁴</u>	<u>Component Code⁵</u>	<u>Cause & Corrective Action to Prevent Recurrence</u>
92-04	920724	S	0	H	5	N/A	ZZ	ZZZZZZ	80% power reduction requested by the System Dispatcher.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training &
License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation
5-Load Reduction
9-Other

4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)
5
Exhibit I - Same Source

DATE: July, 1992

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 1
2. Scheduled date for next refueling shutdown. September 17, 1993
3. Scheduled date for restart following refueling. November 12, 1993
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
Yes, Technical Specification change to increase fuel enrichment from 3.5% to 4.1%.
5. Scheduled date(s) for submitting proposed licensing action and supporting information. The Technical Specification change request was submitted to the NRC on June 27, 1991 (ICAN069108).
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
None.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 625
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
present 968 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
DATE: 1995 (Loss of fullcore offload capability)