UNION ELECTRIC COMPANY CALLAWAY PLANT

MAILING ADDRESS: P. O. BOX 620 FULTON, MO. 65251

November 8, 1984

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Mr. James G. Keppler
Regional Administrator
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

ULNRC-970

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-25
SPECIAL REPORT
EMERGENCY CORE COOLING SYSTEM ACTUATION

Dear Mr. Keppler:

This Special Report is submitted pursuant to Technical Specification 3.5.2 and 6.9.2 concerning an Emergency Core Cooling System actuation.

On August 3, 1984, Callaway Plant Unit 1 experienced an inadvertent Safety Injection (SI). Plant configuration just prior to the event was Mode 4, with the Reactor Coolant System at 385 psig and $326^{\rm O}F$.

The event was initiated when a Instrument and Controls technician placed a Reactor Coolant System (RCS) pressure channel in the TEST mode during the performance of a Technical Specification surveillance test. Three of the four instrument channels that monitor RCS pressure feed a permissive circuit which permits blocking of the Pressurizer Low Pressure/Steamline Low Pressure Safety Injection signals at low RCS pressures. Prior to the event this permissive

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circuit was in the "BLOCK" mode due to the plant being in Mode 4 with low RCS pressure. One of the three redundant pressure channels had failed in the tripped state due to incorrect strapping on the associated bistable. This tripped condition went undetected due to failed lamps on the associated windows of the Partial Trip Status Panel. Thus, when the pressure loop being calibrated was placed in the TEST mode, the required 2 out of 3 coincidence logic for the permissive circuit was satisfied. This caused the block of the SI signal to be automatically reset allowing the Pressurizer Low Pressure SI signal to cause an Emergency Core Cooling System (ECCS) actuation. This is the first ECCS actuation Callaway has experienced at temperature.

To prevent recurrence, the failed lamps in the Partial Trip Status Panel have been replaced and a design change implemented to correct the false bistable output. In addition, I & C procedures are being revised to include a step to require a lamp test on the Partial Trip Status panel prior to putting an instrument channel into test.

Although an Unusual Event was declared, at no time did plant conditions pose a threat to the public health or safety.

S. E. Miltenberger

WRC

SEM/WRC/SMB/1dj

cc: USNRC Resident Inspector, Callaway Missouri Public Service Commission