



Corbin A. McNeill
Senior Vice President
Nuclear Generation

March 15, 1985
JPN-85-21

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Mr. Domenic B. Vassallo
Operating Reactors Branch No. 2
Division of Licensing

Subject: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Appendix R to 10 CFR 50
Request for Scheduler Exemption

References: 1. USNRC Information Notice No. 85-09, "Isolation
Transfer Switches and Post-Fire Shutdown
Capability" dated January 31, 1985

Dear Sirs:

IE Information Notice No. 85-09 (Reference 1) describes a condition at Kansas Gas and Electric Company's Wolf Creek nuclear power plant that could disable the plant's alternate shutdown system in the event of a fire in the control room. Fire damage could open fuses rendering the equipment inoperable if the fuse opens before control is transferred to the alternate shutdown circuit.

As a result of an independent, third party review of the fire protection programs and systems, the Authority has recently identified a similar condition at our FitzPatrick plant. Specifically, the scheme used to transfer control of shutdown systems to the alternate shutdown system does not include redundant fuses.

The Authority has analyzed this condition. This analysis showed that the operators could successfully recover by manually replacing the blown fuse(s). As a result, we will implement interim compensatory measures until long-term corrective actions are completed. Attachment 1 to this letter describes our analysis, the interim compensatory measures and the corrective modifications.

Because this situation was only identified recently, the Authority will not be able to complete the long-term modifications described in Attachment No. 1 within the schedule established by 10 CFR 50.48 for the FitzPatrick plant.

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Therefore, in accordance with the provisions of 10 CFR 50.12 (Specific exemptions, the Authority hereby requests that a schedular exemption be granted from the requirements of Section III.G of Appendix R to 10 CFR 50 to the extent required to implement the modifications described in Attachment No. 1 to this letter. The Authority will complete the modifications described in Attachment No. 1 prior to restart from the next outage of sufficient duration; but not later than restart from the next (Reload 7/Cycle 8) refueling outage which is currently scheduled to begin October 1986 and last approximately 60 days. Compensatory measures described in Attachment No. 1 will be implemented prior to startup from the Reload 6/Cycle 7 refueling outage now in progress.

If you have any questions regarding this issue, please contact Mr. J.A. Gray, Jr. of my staff.

Very truly yours,



Corbin A. McNeill, Jr.
Senior Vice President
Nuclear Generation

cc: U.S. Nuclear Regulatory Commission
Office of the Resident Inspector
P.O. Box 136
Lycoming, New York 13093