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August 10, 1992
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U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for July 1992

Enclosed are two copies of the July 1992 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Broughton
Vice President and Director, TMI-1

WGH:

Attachments

cc: Administrator, Region I
TMI Senior Resident Inspector

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PDR ADOCK 05000289
R PDR

OPERATIONS SUMMARY
JULY 1992

The unit entered the month operating at 100% power producing 838 MWe. It continued full power operation for the entire month of July, closing the month producing 835 MWe. An INPO team was on site for two weeks to perform an evaluation of plant activities.

On July 16th, an underground fuel oil leak was detected via a routine sample from site monitoring wells. The leak was found and the repair effort completed by July 22nd. The entire line was leak tested satisfactorily.

Both a "7 kv Bus Ground" and a "7 kv Bus Trouble" alarm were received in the control room on July 25th for the "B" 7kv bus. Smoke from the associated bus duct was reported and all station loads were transferred to the "A" auxiliary transformer. The "A" diesel was started to satisfy an 8 hour Technical Specification (TS) time clock. A TS 30 Day time clock was subsequently entered as a result of having only one auxiliary transformer in service. The problem resulted from a ground on the "B" phase of the bus resulting from water intrusion from the outside duct in combination with the degradation of the insulation on the bus bar. After repairs were made to the bus bar insulation and the duct was sealed, all loads were realigned. At 0430 on July 26th, the TS time clock was terminated.

NRC Chairman Selin toured TMI-1 on July 27th. Feedback on the tour was positive.

MAJOR SAFETY RELATED MAINTENANCE

During July, the following major safety related maintenance activities were performed:

Miscellaneous Waste Transfer Pump WDL-P-7B

Miscellaneous Waste Transfer Pump WDL-P-7B was removed from service to repair mechanical seal leakage. The Pump was disassembled and inspected. Due to wear, the pump shaft, bearings, oil seals, and oil slinger were replaced. The reassembled pump was fitted with a new mechanical seal, aligned and coupled to the motor. WDL-P-7B was returned to service after it was satisfactorily tested.

Air Intake Tunnel Fire System

During a thunderstorm, a portion of the Air Intake Tunnel Fire Suppression System actuated. Lightning actuated six inside curtain wall halon bottles and set off the Grinnell deluge system. Investigations revealed that ultra-

violet detector 41H was full of water and pressure detectors BS-42A, BS-42D, BS-42E and BS-42F were grounded. The ultra-violet detector and pressure detectors were removed, cleaned, dried, and tested. The six (6) halon bottles were replaced and ultra-violet detector and pressure detectors installed. Functional testing was performed and the system returned to service.

Auxiliary Transformed "B"

A "7 kv Bus Ground" and a "7 kv Bus Trouble" were received on the plant alarm panel. The plant computer indicated that the problem was on the "B" 7 kv bus. Targets were found showing on the ground and 27-2 undervoltage relays. Operators observed smoke entering the ventilation slots in the turbine building wall near the "B" Auxiliary Transformer (AT). The electric load was switched to the "A" AT and the "B" AT was isolated from the 6.9kv bus. While meggering the 6.9kv bus, the "B" phase was found to be grounded. The bus duct was disassembled on both sides of the Turbine Building wall and inspected. The inspection revealed that the heat shrink tubing was burned or eroded away from the bottom of the "B" phase bus bar. Voltage had tracked between two layers of insulating board, which acts as a fire stop/air seal inside the bus ducts housing, to ground via a bolt which holds the insulating board in place. It appeared that moisture had seeped into the duct between the two layers of fiberglass, allowing the voltage to arc from the bus bar to the bolt. The heat shrink tubing was replaced with HVBT tape rated at 15 kv and the insulating board replaced with a single layer of polyethylene. The duct, a short crack in the concrete wall and the gap between the barrier board and the bus bars was sealed to prevent the entry of moisture. The "B" AT was satisfactorily tested and returned to service.

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE August 10, 1992
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: JULY 1992
- 3. LICENSED THERMAL POWER: 2568
- 4. NAMEPLATE RATING (GROSS MWe): 871
- 5. DESIGN ELECTRICAL RATING (NET MWe): 819
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	5111.0	157032.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	5111.0	81842.0
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2245.6
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	5111.0	80767.2
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1908743	13109126	196978172
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	619532	4348238	66342497
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	584736	4105484	62262809
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	51.4
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	51.4
21. UNIT CAPACITY FACTOR (USING MDC NET)		100.0	102.2	50.4
22. UNIT CAPACITY FACTOR (USING DER NET)		96.0	98.1	48.4
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	42.9
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60648.7
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE August 10, 1992
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: JULY

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	785	17	784
2	794	18	782
3	797	19	784
4	793	20	783
5	791	21	785
6	791	22	791
7	794	23	789
8	790	24	791
9	778	25	796
10	783	26	789
11	780	27	780
12	781	28	790
13	776	29	786
14	776	30	785
15	777	31	780
16	784		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July 1992

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE August 10, 1992
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

No.	Date	Type	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code * & *	Component Code * & *	Cause & Corrective Action to Prevent recurrence
						NONE			

1
 F Forced
 S Scheduled

2
 Reason
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

4
 Exhibit G - Instructions for preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1 same source

6 Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
2. Scheduled date for next refueling shutdown: September 17, 1993 (10R)
3. Scheduled date for restart following current refueling: NA
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? NA

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

None planned.

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

GPU Nuclear has installed four Westinghouse Lead Test Assemblies during the reload of the TMI-1 core for cycle 9 operation. Westinghouse fuel technology will be utilized to the extent possible.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 521

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1496. Reracking of spent fuel pool 'A' to attain the licensed capacity is in progress.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The DR (1991) refueling discharge was the last to allow full core off-load capacity (177 fuel assemblies). Upon completion of the reracking project, full core off-load is assured through the end of the current operating license and beyond.