



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

August 13, 1992

Docket Nos. 50-348
and 50-364

LICENSEE: Southern Nuclear Operating Company, Inc.

FACILITY: Joseph M. Farley Nuclear Plant, Units 1 and 2

SUBJECT: SUMMARY OF JUNE 15, 1992, MEETING WITH SOUTHERN NUCLEAR OPERATING
COMPANY, INC.

A management meeting was held with Southern Nuclear Operating Company, Inc. (SNC), on June 15, 1992, in Rockville, Maryland. The purpose of the meeting was to discuss the current restrictions contained in the Joseph M. Farley Nuclear Plant, Units 1 and 2 (Farley), A.C. electrical system Technical Specification (TS). Other subjects of concern were also discussed. Attendees at the meeting were as follow:

NRC

S. Varga
G. Lainas
E. Adensam
S. Hoffman
F. Cantrell
J. Wechselberger

SNC

J. Woodard
D. Morey
C. Nesbitt
B. Moore

On May 7, 1992, the Nuclear Regulatory Commission (NRC) staff sent a letter to SNC indicating that the staff had concerns with the latitude that currently exists in Farley TS 3/4.8.1, A.C. Sources. The staff requested that SNC examine the Farley TS using Westinghouse Standard TS 3/4.8.1, Revision 4A (STS), as a guide, and meet with the staff to explore potential changes. The June 15, 1992, meeting was held in response to the May 7, 1992, letter.

The NRC staff indicated that of particular concern are the following provisions contained in TS 3/4.8.1:

- A. Action Statements 3.8.1.1(b) and (c) and the accompanying footnote allow a diesel generator to be considered operable while it is out of service for 10 days during yearly scheduled maintenance.
- B. Action Statements 3.8.1.1(a) and (b) indicate that the provisions of TS 3.0.4 are not applicable, allowing mode changes with an inoperable diesel generator or offsite circuit.

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- C. Action Statements 3.8.1.1(a) and (b) allow 7 days and 18 days, respectively, to restore an inoperable offsite circuit or diesel generator set. The current STS allows only 72 hours.

To the NRC's knowledge, the full extent of the latitude available in TS 3/4.8.1 has not yet been utilized by SNC, but the latitude exists and could be used in the future. The staff considers this to be a significant issue. In response to the staff's concerns, SNC provided a discussion of their operating philosophy and their understanding of the bases for the current TS. SNC feels that additional latitude was originally justified by the capabilities of Farley's five diesel generator arrangement. However, SNC acknowledged the staff's concerns and agreed to evaluate the current Farley TS 3/4.8.1 with respect to the restrictions contained in the STS. It was agreed that another meeting would be scheduled in approximately 45 days to permit SNC to present the results of their evaluation and any proposals to resolve the staff's concerns. (This meeting was subsequently held on July 31, 1992.)

Additional topics discussed were as follow:

A. Emergency Response Facility Staffing Times

In previous discussions, the staff has expressed concern with provisions in the current Farley emergency plan that allow up to two hours to staff the Technical Support Center and up to four hours to staff the Emergency Operating Facility. This exceeds the NRC guidance of normally staffing within one hour. SNC indicated that they realize that they exceed the normal staffing times, and, although they feel that their emergency plan is adequate, realize the necessity of complying with NRC guidance in this area. SNC is currently working on new concepts for revising their emergency plan and reducing the staffing times. A meeting will be arranged in the future to allow SNC to present the results of their review.

B. Steam Generator Tube Alternate Plugging Criteria

A request for additional information is being prepared with respect to SNC's requested amendments for steam generator tube alternate plugging criteria for the tube support plate region. In a letter dated June 5, 1992, the staff transmitted a set of questions to the Electric Power Research Institute/Steam Generator Reliability Project (EPRI/SGRP) concerning alternate plugging criteria. SNC questioned whether, since Farley is the industry lead plant submitter, they should be responding

to the EPRI/SGRP questions for Farley. The staff indicated that if responses to any of the EPRI/SGRP questions are required for review of Farley's amendment requests, they will be included in a Farley specific request for additional information.

C. Farley Operating Events

During the last Unit 2 refueling outage ending in May 1992, a number of reactor trips occurred, as well as occurrences of manipulating the wrong unit's equipment. A number of events and trips also occurred during the previous two outages at Farley that were discussed at a meeting on September 12, 1991, in the Region II office. For the Unit 2 outage, SNC described the events, their evaluation of their performance, their assessment of the causes of the events, and actions they have taken or are taking. The staff will continue to monitor SNC's actions in this area.

The staff considered the meeting to be beneficial in continuing its discussion and resolution with SNC of issues of importance.

Original signed by:

Stephen T. Hoffman, Project Manager
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Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

cc: See next page

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Joseph M. Farley Nuclear Plant

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