



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY

FERMI-2

DOCKET NO. 50-341

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85  
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Detroit Edison Company (the licensee) dated January 30, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 85, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance with full implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 31, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE

3/4 4-11\*  
3/4 4-12

INSERT

3/4 4-11\*  
3/4 4-12

\*Overleaf page provided to maintain document completeness. No changes are contained in this page.

## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS

4.4.3.2.1 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- a. Monitoring the primary containment atmospheric gaseous radioactivity at least once per 4 hours,\*
- b. Monitoring the primary containment sump flow rate at least once per 4 hours,
- c. Monitoring the drywell floor drain sump level at least once per 4 hours, and
- d. Monitoring the reactor vessel head flange leak detection system at least once per 24 hours.\*

4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE by leak testing pursuant to Specification 4.0.5 and verifying the leakage of each valve to be within the specified limit:

- a. At least once per 18 months, and
- b. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate.

The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3.

4.4.3.2.3 The high/low pressure interface valve leakage pressure monitors shall be demonstrated OPERABLE with alarm setpoints per Table 3.4.3.2-2 by performance of a:

- a. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
- b. CHANNEL CALIBRATION at least once per 18 months.

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\*Not a means of quantifying leakage.

TABLE 3.4.3.2-1  
REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>VALVE DESCRIPTION</u>
1. RHR System	
E11-F015A	LPCI Loop A Injection Isolation Valve
E11-F015B	LPCI Loop B Injection Isolation Valve
E11-F050A	LPCI Loop A Injection Line Testable Check Valve
E11-F050B	LPCI Loop B Injection Line Testable Check Valve
E11-F008	Shutdown Cooling RPV Suction Outboard Isolation Valve
E11-F009	Shutdown Cooling RPV Suction Inboard Isolation Valve
E11-F608	Shutdown Cooling Suction Isolation Valve
2. Core Spray System	
E21-F005A	Loop A Inboard Isolation Valve
E21-F005B	Loop B Inboard Isolation Valve
E21-F006A	Loop A Containment Check Valve
E21-F006B	Loop B Containment Check Valve
3. High Pressure Coolant Injection System	
E41-F007	Pump Discharge Outboard Isolation Valve
E41-F006	Pump Discharge Inboard Isolation Valve
4. Reactor Core Isolation Cooling System	
E51-F012	Pump Discharge Isolation Valve
E51-F013	Pump Discharge to Feedwater Header Isolation Valve

TABLE 3.4.3.2-2  
REACTOR COOLANT SYSTEM INTERFACE VALVES  
LEAKAGE PRESSURE MONITORS

<u>VALVE NUMBER</u>	<u>SYSTEM</u>	<u>ALARM SETPOINT (psig)</u>
E11-F015A & B, E11-F050A & B	RHR LPCI	≤ 449
E11-F008, F009, F608	RHR Shutdown Cooling	≤ 135
E21-F005A & B, E21-F006A & B	Core Spray	≤ 452
E41-F006, F007	HPCI	≤ 71
E51-F012, F013	RCIC	≤ 71