NRC Form 366 (9-83)					LIC	LICENSEE EVENT REPORT (LER)							U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/86										
EACH ITY	NAME (1	1	-			-	-	-				-				Tpoc	KET	NUM	BER (2)		PA	AGE (3)
Frowns Ferry - Units 1,						2	and 3						0 15 10 10 1						10	F 0 12			
TITLE (4)		2 16	I I y		OHL	00	+ ,	۷,	and							10	10	0	0	0 1	4 3 3	1.10	To In
					04-		- 0	n.	1	0.0				3 01	nd D								
	_		rte	-	-	-	-	חו		-	-	-	ors (, a									
EVENT DATE (6) LER NUMBER (6)						REPORT DATE (7)				OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR SEQUENTIAL NUMBER				, A	EVISION IUMBER	MONTH	DAY	YI	YEAR		FACILITY NA							T NUMBE		
													Brow	ns	Ferry	7 -	Un	it	2	0 15	1010	1012	21610
						. 1						. [
0 2	0 5	8 5	8 5		0 0	3		0 0	0 3	0 5	5 8	5	Brow	ns !	Ferry	7 -	Un	it	3	0 5	1010	1012	21916
005	DATINO		THIS RE	PORT	IS SUBM	TTED	PURI	SUAN' 1	TO THE R	EQUIRE	MENT	8 OF 10	CFR 8: 10	Check o	ne or mor	e of th	ne follo	owing	(11)				
MODE (9)			20.402(b)					20.408(c) X 50.73(a)(2)(iv					3(a)(2)(iv)	73.71(6				3,71(b)	(b)				
POWER 20.405(a)(1)(i)						50.36(c)(1) 50.73(a)(2)(v)						73.71(e)											
LEVEL (10) 0 10 10 20.408(a)(1)(iii				50.36(e)(2)						50.73(a)(2)(vii)				1	OTHER (Specify in Abstract								
			20.406(a)(1)(iii)						50.73(a) (2) (4) 50.73(a) (2) (viii						(A)				below end in Text, NRC Form 368AI				
								-					-	-						,	00A/		
\Box				20.405(a)(1)(lv)					50.73(a)(2)(ii) 50.73(a)(2)(viii						(8)								
			20	1,406(a)	(1)(v)				50.73(a)	(2)(iii)				50.73	3(a)(2)(x)								
								L	ICENSEE	CONTAC	CT FO	R THIS	LER (12)										
NAME																				ELEPH	ONE NUM	BER	
																	AREA	A CO	DE				
St	eve J	ones															21	01	5	712	2191-	12 15	13 18
					COMPLE	ETE O	NE LI	NE FOR	EACH CO	MPONE	NT FA	ILURE	DESCRIBE	DINT	HIS REP	DAT (1	-						
		-		1	ANUFAC	-	BERGE	TABLE								T			~	T			
CAUSE	SYSTEM	COMPO	NENT TURER		- 1	TO NPROS					CAUSE	SYSTEM COMPONENT			MANUFAC- TURER				NPRDS				
				+		-					-		-	-		+	-	_		+			*******
					. 5. 5																		
_			ш.	\vdash		-		-							11	+	_	1	_	-			i
				1																			
															11			1	1				
SUPPLEMENTAL REPORT EXPECTED (14)												EXP	ECTED		MONTH	DAY	YEAR						
																	S	UBM	1\$\$10	N		-	
10000	111 vac o	omplete E	XPECTED	SUBA	HISSION D	ATEL			X	NO						- 10		UAT	2 (19)				1 1

Common diesel generators C and D for units 1 and 2 started during functional testing of protective relays for the unit 2 station transformers A and B, unit 2 main transformer, and unit 2 generator. The initiations were the result of the 250 volt DC battery board 4 bus filter breaker being left open, allowing a voltage spike to trip the 161 KV lines.

9503140520 850305 PDR ADDCK 05000259 S PDR

NRC Form 366

IF27

PACILITY NAME (1)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION								
		LER NUMBI		PAGE (3)					
Browns Ferry - Units 1, 2, and 3 TEXT (N more space is required, use additional NRC form 386A 2) (17) Unit 1 was operating at 08 popularity	0 5 0 0 0 2 5 9	8 5 -0 0	REVISION NUMBER	12 OF 0 1 2					

Unit 1 was operating at 98 percent power, unit 2 was in a refueling outage, and unit 3 was at 100 percent power. All three units were affected by the event.

On February 5, 1985, during functional testing of the protective relays (RLY) for the unit 2 station transformer A and B (XFMR), unit 2 main transformer (XFMR), and unit 2 the unit 2 generator breaker (BKR) control circuit. The voltage spike traveled to sensitive cocling tower transformer overpressurization relays (RLY) because breaker 211 voltage spike caused the cooling tower transformers overpressurization relays (RLY) because breaker 211 voltage spike caused the cooling tower transformers overpressurization relays (RLY) to boards which initiated diesel generators C and D (DG). The diesels started but did not room where the licensed reactor operator verified no unusual event was present. The

During the event, the safety functions of the diesel generators were unaffected. Therefore, no hazard existed to any safety related equipment.

The voltage spike affected the overpressurization relays because the 250 volt DC battery board 4 bus filter was inoperative. This inoperability was caused by breaker 211 being left open after Operations Section Instruction Letter (OSIL) 51, locating DC grounds, was performed. Caution signs have been installed next to breaker 211 on battery boards, as well as the breakers on the other battery boards. OSIL-51 is being revised to include a caution on leaving the breaker open and to coordinate with the Division of replacing or desensitizing the overpressurization relays in order to reduce the probability of voltage spikes from occurring in the future.

Responsible Plant Section - EM and OP

Previous Events - None

TENNESSEE VALLEY AUTHORITY Browns Ferry Nuclear Plant P. 0. Box 2000 Decatur, Alabama 35602

March 5, 1985

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 1 - DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE REPORT BFR0-50-259/85003

The enclosed report provides details concerning the inadvertent start of diesel generators C and D. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones Plant Manager

Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

NRC Resident Inspector, BFN

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

IE22