

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON D. C. 20555

THE UNIVERSITY OF UTAH

DOCKET NO. 50-407

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6 License No. R-126

- The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - The application for amendment to Facility Operating License No. R-126 filed by the University of Utah (the licensee), dated May 20, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - F. Prior notice of this amendment was not required by 10 CFR 2.105(a)(4) and publication of notice for this amendment is not required by 10 CFR 2.106(a)(2).

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. R-126 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as relised through Amendment No. 6, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Seymour H. Weiss, Director

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Non-Power Reactors, Decommissioning, and Environmental Project Directorate Division of Reactors Projects - III/IV/V

Office of Nuclear Reactor Regulation

Enclosure: Appendix A Technical Specifications Changes

Date of Issuance:

FACILITY OPERATING LICENSE NO. R-126 DOCKET NO 50-407

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove Page

Insert Page

19

insures the same degree of control of release of radinactive materials (SAR Section 8.7.5).

4.3.5 Experiment and Irradiation Limits

Applicability: This specification applies to the surveillance requirements for experiments installed in the reactor and its experimental facilities and for irradiations performed in the irradiation facilities.

Specifications:

- (1) A new experiment shall not be installed in the reactor or its experimental facilities until a hazards analysis has been performed by the Reactor Supervisor and reviewed by the Reactor Safety Committee. Minor modifications to reviewed and approved experiments may an made at the discretion of the senior operator responsible for the reviewed at the hazards associated with the material and the been reviewed and a determination has been and that the fications do not create a significantly do the part of a greater hazard than the original approved experiment.
- (2) An irradiation of a new type of device or material shall not be performed until an analysis of the irradiation has been performed and reviewed by the Reactor Supervisor.

Basis: It has been demonstrated over a number of years that experiments and irradiations reviewed by the reactor staff and the Reactor Safety Committee, as appropriate, can be conducted without endangering the safety of the reactor or exceeding the limits in the Technical Specifications.

4.4 Reactor Fuel Elements

Applicability: This specification applies to the surveillance requirements for the fuel elements.

Objective: The objective is to verify the continuing integrity of the fuel element cladding.

Specifications: All fuel elements shall be inspected visually for damage or deterioration every two years. Any fuel element which appears damaged shall be measured for length and bend. The reactor shall not be operated with damaged fuel. A fuel element shall be considered damaged and must be removed from the core if:

- (1) in measuring the transverse bend, its sagitta exceeds 0.125 inches over the length of the cladding,
- (2) in measuring the elongation, its length exceeds its original length by 0.250 inches,