



Commonwealth Edison  
LaSalle County Nuclear Station  
2601 N. 21st. Rd.  
Marseilles, Illinois 61341  
Telephone 815/357-6761

August 10, 1992

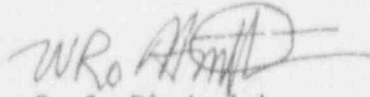
Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering  
LaSalle County Nuclear Power Station for July 1992.

Very truly yours,

  
G. J. Diederich  
for Station Manager  
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION  
UNIT 1  
MONTHLY PERFORMANCE REPORT  
JULY 1992  
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 79-373

LICENSE NO. NPF-11

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(UNIT 1)

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## I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 850 Mwe.
	1100	Increased power level to 1095 Mwe (Unit in coastdown).
3	0100	Reduced power level to 850 Mwe due to system load.
	1200	Increased power level to 1100 Mwe.
	1900	Reduced power level to 1000 Mwe due to system load.
	2300	Reduced power level to 850 Mwe due to system load.
4	1300	Increased power level to 1090 Mwe.
	2000	Reduced power level to 850 Mwe due to system load.
5	1100	Increased power level to 1060 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
6	1400	Increased power level to 1090 Mwe.
	2000	Reduced power level to 850 Mwe due to system load.
7	1200	Increased power level to 1075 Mwe.
	1430	Reduced power level to 975 Mwe due to trip of the '1C' Circulating Water Pump. Pump trip was caused by high differential pressure across the suction screen.
	1630	Increased power level to 1070 Mwe.
8	0100	Reduced power level to 840 Mwe due to system load.
	1200	Increased power level to 1090 Mwe.
	2100	Reduced power level to 840 Mwe due to system load, placed the '1A' Turbine Driven Reactor Feed Pump out of service for maintenance.
9	0800	Increased power level to 1010 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
10	0630	Increased power level to 990 Mwe.
	1900	Increased power level to 1080 Mwe, placed the '1A' Turbine Driven Reactor Feed Pump back in service.
11	0430	Reduced power level to 900 Mwe due to system load.

## II. MONTHLY REPORT (CONTINUED)

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
	0800	Increased power level to 1080 Mwe.
12	0030	Reduced power level to 850 Mwe due to system load.
	1100	Increased power level to 1075 Mwe.
13	0330	Reduced power level to 850 Mwe due to system load.
14	1100	Increased power level to 1070 Mwe.
15	2330	Reduced power level to 850 Mwe due to system load.
16	0700	Increased power level to 1075 Mwe.
17	0030	Reduced power level to 850 Mwe due to system load.
18	0100	Reduced power level to 830 Mwe due to system load.
	0700	Increased power level to 1050 Mwe.
	2300	Reduced power level to 850 Mwe due to system load.
19	0730	Increased power level to 1050 Mwe.
20	0100	Reduced power level to 950 Mwe due to system load.
	0530	Increased power level to 1045 Mwe.
22	0400	Reduced power level to 1030 Mwe and held for TIP set.
	2335	Reduced power level to 850 Mwe due to system load.
23	0530	Increased power level to 1030 Mwe.
	2130	Reduced power level to 760 Mwe due to system load.
24	0600	Increased power level to 1030 Mwe.
	2300	Reduced power level to 850 Mwe due to system load.
25	0730	Increased power level to 1030 Mwe.
26	0100	Reduced power level to 750 Mwe due to system load.
	1600	Increased power level to 850 Mwe.
	2130	Reduced power level to 780 Mwe due to system load.

## II. MONTHLY REPORT (CONTINUED)

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
27	0430	Increased power level to 1030 Mwe.
	2130	Reduced power level to 750 Mwe due to system load, performed monthly surveillances.
28	0700	Increased power level to 1030 Mwe.
	2300	Reduced power level to 770 Mwe due to system load.
29	0610	Increased power level to 1020 Mwe.
	2100	Reduced power level to 850 Mwe due to system load.
30	0630	Increased power level to 1020 Mwe.
	1700	Reduced power level to 890 Mwe due to system load.
	2130	Reduced power level to 760 Mwe due to system load.
31	0700	Increased power level to 1010 Mwe.
	1230	Increased power level to 1025 Mwe.
	2300	Reduced power level to 850 Mwe due to system load.
	2400	Reactor critical, Generator on-line at 780 Mwe, continuing coastdown. Power level reduced due to system load.

- B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION  
(None.)
  
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including  
SOR differential pressure switch failure reports).  
(See Table 1)
  
- D. LICENSEE EVENT REPORTS (Unit 1)  
(None.)
  
- E. DATA TABULATIONS (Unit 1)
  - 1. Operating Data Report (See Table 2)
  - 2. Average Daily Unit Power Level (See Table 3)
  - 3. Unit Shutdowns and Significant Power Reductions (See Table 4)



## C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L16309	Control Room HVAC Ammonia Detector OXY-VC165A	Moisture in flow meter.	No indication.	Purged sensing lines and replaced flow meter.
L16401	Control Room HVAC Return Fan OVC02CB	Grounded motor windings.	Fan tripped on thermal overload.	Replaced motor.
L16474	Switchgear Heat Removal Vent Fan TVX04C	Fan supply breaker.	Fan supply breaker tripped on magnetics.	Replaced breaker.
L16815	Hydraulic Control Unit 50-35	Nitrogen charging nipple leaking.	Degraded operation.	Replaced charging nipple.

(No SOR Failures this month.)

TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE August 10, 1992  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: July 1992  
GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323  
MAX DEPENDABLE CAPACITY (MWe-Net): 1,036  
DESIGN ELECTRICAL RATING (MWe-Net): 1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASON FOR RESTRICTION (IF ANY):

	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	5,049.5	52,406.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	744.0	5,016.8	51,395.1
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh <sub>t</sub> )	2,204,697	15,930,868	151,087,347
10. ELECTRICAL ENERGY GENERATED (MWh <sub>e</sub> -Gross)	732,090	5,382,090	50,471,865
11. ELECTRICAL ENERGY GENERATED (MWh <sub>e</sub> -Net)	705,907	5,208,836	48,403,981
12. REACTOR SERVICE FACTOR (%)	100.0	98.8	69.6
13. REACTOR AVAILABILITY FACTOR (%)	100.0	98.8	71.8
14. UNIT SERVICE FACTOR (%)	100.0	98.2	68.3
15. UNIT AVAILABILITY FACTOR (%)	100.0	98.2	68.3
16. UNIT CAPACITY FACTOR (USING MDC) (%)	91.6	98.4	62.1
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	88.0	94.5	59.7
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	1.8	7.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
Refuel Overhaul, 09/26/92, 13 Weeks

20. IF SHUTDOWN DURING REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE August 10, 1992  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: July 1992

DAY	POWER	DAY	POWER
1	991	17	961
2	1,072	18	963
3	954	19	951
4	925	20	997
5	928	21	1,004
6	936	22	1,002
7	973	23	951
8	945	24	922
9	931	25	941
10	941	26	790
11	1,002	27	925
12	970	28	894
13	994	29	903
14	989	30	896
15	961	31	875
16	942		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%  
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
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(None.)

## SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations  
(None.)
2. ECCS System Outages  
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual  
(None.)
4. Major changes to Radioactive Waste Treatment Systems.  
(None.)
5. Indications of Failed Fuel Elements.  
(None.)

(Unit 1)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
0550	ODG01K	Replace fuel filter.
0552	ODG01K	Replace soakback oil filter and clean oil strainer.
0557	ODG023B	Disassemble and inspect check valve.
(U-1)		
0536	1E22-S001	Inspect HACR relay test switches.
0537	1E22-C302B	Air compressor inspection.
0556	1E12-C003	Megger motor.
0557	1E12-C003	Coupling lubrication.
0560	1E12-F006B	Inspection, refurbishment, and
0561		VOTES test.
0562		
0566		
0573	1E12-F051B 1E12-F065B	Administrative control.
0596	1DG035	Administrative control.

LASALLE NUCLEAR POWER STATION  
UNIT 2  
MONTHLY PERFORMANCE REPORT  
JULY 1992  
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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- F. UNIQUE REPORTING REQUIREMENTS
  - 1. Safety/Relief Valve Operations
  - 2. ECCS System Outages
  - 3. Off-Site Dose Calculation Manual Changes
  - 4. Major Changes to Radioactive Waste Treatment System
  - 5. Indications of Failed Fuel Elements



## I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 900 Mwe.
3	2300	Reduced power level to 850 Mwe due to system load.
4	0700	Increased power level to 1080 Mwe.
	1900	Reduced power level to 850 Mwe due to system load.
5	1500	Increased power level to 1075 Mwe.
	2400	Reduced power level to 875 Mwe due to system load.
6	1130	Increased power level to 1100 Mwe.
	1400	Reduced power level to 1065 Mwe due to APRM high alarms.
	2200	Reduced power level to 850 Mwe due to system load.
	1400	Increased power level to 1105 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
8	1300	Increased power level to 1125 Mwe.
10	0100	Reduced power level to 850 Mwe due to system load.
	1200	Increased power level to 1125 Mwe.
11	0130	Reduced power level to 990 Mwe due to system load.
	1100	Increased power level to 1115 Mwe.
12	0230	Reduced power level to 850 Mwe due to system load.
	1300	Increased power level to 1115 Mwe.
15	0200	Reduced power level to 850 Mwe due to system load.
	1200	Increased power level to 1100 Mwe.
	2300	Reduced power level to 850 Mwe due to system load.
16	1300	Increased power level to 1090 Mwe.
17	1300	Reduced power level to 990 Mwe due to loss of the Main Power transformer fans.
	1320	Increased power level to 1100 Mwe.
18	2300	Reduced power level to 850 Mwe due to system load.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2) (CONTINUED)

<u>Day</u>	<u>Time</u>	<u>Event</u>
19	0900	Increased power level to 1090 Mwe.
	2000	Reduced power level to 990 Mwe due to system load.
	2350	Reduced power level to 850 Mwe due to system load.
20	0430	Increased power level to 1090 Mwe.
21	0020	Reduced power level to 850 Mwe due to system load.
	0640	Increased power level to 1085 Mwe.
	2200	Reduced power level to 850 Mwe due to system load.
22	0520	Increased power level to 1090 Mwe.
	2230	Reduced power level to 1040 Mwe due to high Main Turbine bearing vibrations.
25	0220	Reduced power level to 850 Mwe due to system load.
	0600	Increased power level to 1080 Mwe.
26	0530	Reduced power level to 840 Mwe due to system load.
	1400	Increased power level to 1070 Mwe.
27	0000	Reduced power level to 850 Mwe due to system load.
	0830	Increased power level to 1070 Mwe.
28	0010	Reduced power level to 850 Mwe due to system load.
29	0600	Increased power level to 1080 Mwe.
	1500	Reduced power level to 850 Mwe due to system load.
30	0640	Increased power level to 1080 Mwe.
31	0100	Reduced power level to 850 Mwe due to system load.
	0300	Increased power level to 1080 Mwe.
	2400	Reactor critical, Generator on-line at 1080 Mwe.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-009-00	07/14	Failure of the reactor core isolation cooling isolation valve 2E51-F086.

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.  
(See Table 2)
2. Average Daily Unit Power Level.  
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.  
(See Table 4)

## C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L16200	250 Volt DC Battery 2DC01E	250V DC battery discharge alarm unit.	Discharge high alarm indication.	Recalibrated unit.
L16639 L16678	Residual Heat Removal Test Return Valve 2E12-F024A	Motor feed cable insulation cut by MCC door.	MCC feed breaker tripped on magnetics when valve was opened.	Replaced feed cable. Replaced anti-rotation locking key.
L91670	Reactor Building Ventilation Pipe Tunnel Inlet Stop Damper 2VR10Y	Constant force springs.	Damper failed to close.	Replaced springs.

(No SOR Failures this month.)

TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LASALLE TWO  
DATE August 10, 1992  
COMPLETED BY M.J.CIALKOWSKI  
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:	July 1992		
GROSS HOURS IN REPORTING PERIOD:	744		
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323		
MAX DEPENDABLE CAPACITY (MWe-Net):	1,036		
DESIGN ELECTRICAL RATING (MWe-Net):	1,078		
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		
4. REASON FOR RESTRICTION (IF ANY):			
	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	2,761.2	47,697.1
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	2,599.4	46,803.9
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWhT)	2,277,885	7,734,657	139,977,056
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	764,308	2,622,632	46,512,382
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	737,508	2,502,580	44,634,084
12. REACTOR SERVICE FACTOR (%)	100.0	54.0	69.9
13. REACTOR AVAILABILITY FACTOR (%)	100.0	54.0	72.4
14. UNIT SERVICE FACTOR (%)	100.0	50.9	68.6
15. UNIT AVAILABILITY FACTOR (%)	100.0	50.9	68.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	95.7	47.3	63.1
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	92.0	45.4	60.7
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	3.8	12.7
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			

E.2 TABLE 3  
AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373  
UNIT LASALLE TWO  
DATE August 10, 1992  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: July 1992

DAY	POWER	DAY	POWER
1	993	17	975
2	1,073	18	1,062
3	1,081	19	948
4	951	20	984
5	914	21	968
6	947	22	973
7	966	23	998
8	1,000	24	995
9	1,068	25	996
10	1,002	26	922
11	1,037	27	968
12	993	28	947
13	1,071	29	959
14	1,061	30	979
15	966	31	975
16	954		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%  
(UNIT 2)

YEARLY SEQUENTIAL DATE NUMBER	(YMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
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(None.)

## SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities.



F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

(None.)

2. ECCS System Outages

(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.

(None.)

4. Major changes to Radioactive Waste Treatment Systems.

(None.)

5. Indications of Failed Fuel Elements.

(None.)

(Unit 2)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
1866	2DG02CA	Air compressor inspection.
1871	2DG08CA	Motor inspection and repair.
1872	2DG09DA	Repair discharge check valve.
1877	2E22-C001	Lubrication.
1881	2E51-C002	Change oil and filters.
1886	2E12-F024A	Replace motor and anti-rotation device.
1887		
1893		
1889	2E51-F080	Repack 2E51-F086.
1913	2E51-F086	
1914		
1917		
1908	2E12-C002C	Install drain lines on oil reservoir.
1909		