Docket No. 50-423
B14104

Attachment 1

Millstone Nuclear Power Station, Unit No. 3
Proposed Revision to Technical Specifications

July 1992

BASIS

ENCLOSURE BUILDING INTEGRITY

1.12 ENCLOSURE BUILDING INTEGRITY shall exist when:

a. Each door in each access opening is closed except when the access opening is being used for normal transit entry and exit,

b. The Supplementary Leak Collection and Release System is OPERABLE, and

c. The sealing mechanism associated with each penetration (e.g., welds, bellows, or O-rings) is OPERABLE.

ENGINEERED SAFETY FEATURES RESPONSE TIME

1.13 The ENGINEERED SAFETY FEATURES (ESF) RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF Actuation Setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable.

1.14 Deleted

FREQUENCY NOTATION

1.15 The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.1.

IDENTIFIED LEAKAGE

1.16 IDENTIFIED LEAKAGE shall be:

a. Leakage (except CONTROLLED LEAKAGE) into closed systems, such as pump seal or valve packing leaks that are captured and conducted to a sump or collecting tank, or

b. Leakage into the containment atmosphere from sources that are both specifically located and known either not to interfere with the operation of Leakage Detection Systems or not to be PRESSURE BOUNDARY LEAKAGE, or

c. Reactor Coolant System leakage through a steam generator to the Secondary Coolant System.

MASTER RELAY TEST

1.17 A MASTER RELAY TEST shall be the energization of each master relay and verification of OPERABILITY of each relay. The MASTER RELAY TEST shall include a continuity check of each associated slave relay.

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REMOTE SHUTDOWN INSTRUMENTATION (Continued)

instrumentation, control, and power circuits and transfer switches necessary to eliminate effects of the fire and allow operation of instrumentation, control and power circuits required to achieve and maintain a safe shutdown condition are independent of areas where a fire could damage systems normally used to shut down the reactor. This capability is consistent with General Design Criterion 3 and Appendix R to 10 CFR Part 50.

3/4.3.3.6 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. The instrumentation included in this specification are those instruments provided to monitor key variables, designated as Category 1 instruments following the guidance for classification contained in Regulatory Guide 1.97, Revision 2, "Instrumentation for Light-Water-Cooled Nuclear Power Plants To Assess Plant and Environs Conditions During and Following an Accident."

3/4.3.3.7 Deleted.

3/4.7.11 SEALED SOURCE CONTAMINATION (Continued)

plutonium. This limitation will ensure that leakage from Byproduct, Source, and Special Nuclear Material sources will not exceed allowable intake values.

Sealed sources are classified into three groups according to their use, with Surveillance Requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism (i.e., sealed sources within radiation monitoring or boron measuring devices) are considered to be stored and need not be tested unless they are removed from the shielded mechanism.

3/4.7.12 Deleted

3/4.7.13 Deleted

3/4.7.14 AREA TEMPERATURE MONITORING

The area temperature limitations ensure that safety-related equipment will not be subjected to temperatures in excess of their environmental qualification temperatures. Exposure to excessive temperatures may degrade equipment and can cause a loss of its OPERABILITY. The temperature limits include an allowance for instrument error of ±2.2°F.

FACILITY STAFF (Continued)

b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room;

c. At least two licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.

d. A Health Physics Technician* shall be on site when fuel is in the reactor;

e. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or licensed Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;

f. Deleted

g. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions. These procedures should follow the general guidance of the NRC Policy Statement on working hours (Generic Letter No. 82-12).

*The Health Physics Technician may be absent for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required position.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1, May 1977. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Station Superintendent and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.

6.4.2 Deleted.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Unit Superintendent on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of the:

Chairman:	Unit Superintendent
Vice Chairman and Member:	Operations Supervisor
Member:	Maintenance Supervisor
Member:	Instrument and Control Supervisor
Member:	Reactor Engineer
Member:	Engineering Supervisor or Startup Supervisor*
Member:	Station Services Superintendent or Quality Services Supervisor or Radiological Services Supervisor
Member:	Staff Engineer**

*When position is staffed.

**The Staff Engineer member of the PORC shall have an academic degree in engineering or physical science field; and, in addition, shall have a five years technical experience, of which a minimum of three years shall be in the nuclear power plant industry.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman.

QUORUM

6.5.1.5 The quorum of the PORC shall consist of the Chairman or Vice Chairman or Station Superintendent and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for:

a. Review of: (1) all procedures, except common site procedures, required by Specification 6.8 and changes thereto, and (2) any other proposed procedures or changes thereto as determined by the Unit Superintendent to affect nuclear safety;

b. Review of all proposed tests and experiments that affect nuclear safety;

c. Review of all proposed changes to Sections 1.0-5.0 of these Technical Specifications;

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;

e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President-Nuclear Operations and to the Chairman of the Nuclear Review Board;

f. Review of all REPORTABLE EVENTS;

g. Review of facility operations to detect potential safety hazards;

h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Chairman of the Nuclear Review Board or the Station Superintendent; and

i. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6a. through d. above constitutes an unreviewed safety question.

j. Review of Unit Turbine Overspeed Protection Maintenance and Testing Program and revision thereto.

k. Review of the Fire Protection Program and implementing procedures

QUORUM

6.5.2.5 A quorum of the SORC shall consist of the Chairman and four members including alternates.

RESPONSIBILITIES

6.5.2.6 The SORC shall be responsible for:

a. Review of (1) all common site procedures required by Specification 6.8 and changes thereto, (2) any other proposed procedures or changes thereto as determined by the Station Superintendent to affect site nuclear safety;

b. Review of all proposed changes to Section 6.0 "Administrative Controls" of these Technical Specifications;

c. Performance of special reviews and investigations and reports as requested by the Chairman of the Site Nuclear Review Board;

d. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the Chairman of the Site Nuclear Review Board;

e. Review of the Emergency Plan and implementing procedures, and submittal of recommended changes to the Chairman of the Site Nuclear Review Board;

f. Review of all common site proposed tests and experiments that affect nuclear safety;

g. Review of all common site proposed changes or modifications to systems or equipment that affect nuclear safety; and

h. Render determinations in writing or meeting minutes with regard to whether or not each item considered under Specification 6.5.2.6(a) through (g) above constitutes an unreviewed safety question.

i. Review of the common site Fire Protection Program and implementing procedures.

AUTHORITY

6.5.2.7 The SORC shall:

a. Recommend to the Station Superintendent written approval or disapproval in meeting minutes of items considered under Specification 6.5.2.6(a) through (g) above, and

b. Provide immediate written notification or meeting minutes to the Vice President-Nuclear Operations and the Chairman of the Site Nuclear Review Board of disagreement between the SORC and the Station Superintendent; however, the Station Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.2.8 The SORC shall maintain written minutes of each meeting and copies shall be provided to the Vice President-Nuclear Operations and Chairman of the Site Nuclear Review Board.

(5) Inservice Inspection Program (Section 5.2.4.3, SER, 6.6.3 SER)

Prior to May 25, 1986, NNECO shall submit the inservice inspection program which conforms to the ASME Code in effect on November 25, 1984 in accordance with 50.55(a)(g)(4), for NRC staff review and approval.

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(7) Deleted

(8) Deleted

(9) Deleted

Docket No. 50-423
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Attachment 2

Millstone Nuclear Power Station, Unit No. 3

Proposed Changes to the Operating License

July 1992

(5) Inservice Inspected Program (Section 5.2.4.3, SER, 6.6.3 SER)

Prior to May 25, 1986, NNECO shall submit the inservice inspection program which conforms to the ASME Code in effect on November 25, 1984 in accordance with 50.55(a)(g)(4), for NRC staff review and approval.

(6) Deleted

(7) Deleted

(8) Deleted

(9) Deleted

follow-up within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).

G. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

H. This license is effective as of the date of issuance and shall expire at midnight on November 25, 2025.

I. Fire Protection (Section 9.5.1, SER, SSER 2, SSER 4, SSER 5)

Northeast Nuclear Energy Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SER (NUREG-1031) issued July 1984 and Supplements Nos. 2, 4, and 5 issued September 1985, November 1985 and January 1986, respectively, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those charges would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by H. R. Denton

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Attachments/Appendices
1. Appendix A - Technical Specifications (NUREG-1176)
2. Appendix B - Environmental Protection Plan

Date of Issuance: January 31, 1986