

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5   0   0   0   3   6   2	PAGE (3) 1 OF 0 4
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TITLE (4)  
DELINQUENT RCS SAMPLE WITH DOSE EQUIVALENT IODINE LIMITS EXCEEDED

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 1	3 0	8 5	8 5	0 0 4	0 0	0 1	2 7	8 5			0 5   0   0   0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											

OPERATING MODE (9) 5	POWER LEVEL (10) 0 0   0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
		<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
		<input type="checkbox"/> 20.405(a)(1)(ii)	<input checked="" type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
		<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
		<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME J. G. HAYNES, STATION MANAGER	TELEPHONE NUMBER
	AREA CODE: 7 1 4    4 9 2   -   7 7 0   0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 30, 1985, at 0900, with Unit 3 in Mode 5, following collapsing of the pressurizer steam bubble after a shutdown, analysis of a Reactor Coolant System (RCS) sample, taken only for observation purposes and not required by Technical Specifications, indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131.

RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by 1930 with purification flow. The increased activity was caused when the iodine in the pressurizer steam bubble was forced into solution while collapsing the bubble.

A 4 hour sample due to be taken at 1650 was not taken until 1930. Our investigation of the administrative controls surrounding the delinquent sample determined that the cause of the delinquent sample was personnel error in that the responsible chemistry technician and chemistry foreman understood the sampling requirements but failed to complete them within the required time. Appropriate disciplinary action was taken. Additionally, this event was discussed with all chemistry technicians and foremen.

This submittal also provides the report pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd', RCS specific activity exceeding 1.0 microcuries/gram Dose Equivalent I-131.

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PDR ADOCK 05000362  
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LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5 1 0 0 0 3 6 2	LER NUMBER (6)			PAGE (3)	
		YEAR 8 5	SEQ. NUMBER - 0 0 4	REV. NUMBER - 0 0	0 2	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On January 30, 1985, at 0900, with Unit 3 in Mode 5, following collapsing of the pressurizer steam bubble after a shutdown, analysis of a Reactor Coolant System (RCS)(EIIS System Code AB) sample, taken only for observation purposes and was not required by Technical Specifications, indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131.

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This submittal also provides the report pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd', Reactor Coolant System (RCS) specific activity exceeding 1.0 microcuries/gram Dose Equivalent I-131.

Additional information, required by LCO 3.4.7, Action Statement 'd', is provided on the following pages. Although the unit has a degassification path which operates continuously and takes pressurizer steam, condenses it, and directs it to Liquid Radwaste, degassing operation history is not applicable, because this system reduces the noble gas content of the Reactor Coolant System but has no effect on iodine.

CLEANUP FLOW HISTORY

<u>PERIOD</u>	<u>AVERAGE CLEANUP FLOW (GPM)</u>
1/28/85, 0900 to 1/30/85, 1930	85*

\*Hourly cleanup flow data not available. Figure used is taken from average flow with two charging pumps in operation.

REACTOR POWER HISTORY

<u>PERIOD</u>	<u>REACTOR POWER</u>
1/28/85, 0900 to 1/30/85, 1930	0%

REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY ANALYSIS

<u>DATE</u>	<u>TIME</u>	<u>DE I-131 MICROCURIES/GRAM</u>
1/30/85	0900	1.41
1/30/85	1250	1.36
1/30/85	1930	0.85

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3

DOCKET NUMBER (2)

0500036285

LER NUMBER (6)

YEAR SEQUENTIAL NUMBER REVISION NUMBER

0100410003 OF 04

PAGE (3)

03 OF 04

TEXT (If more space is required, use additional NRC Form 366A's) (17)

M2 1280

53C1F 032

EDIT= 23 01/21/85 09:43:35

PAGE =

AXIALLY INTEGRATED AND PEAK OUTPUT ASSEMBLY EXPOSURE EDITS

FORMAT OF ASSEMBLY IN CORE MAP

ASSEMBLY NUMBER - SWITCH NUMBER  
INTEGRATED BOX EXPOSURE IN 10\*\*003MWD/T  
MAXIMUM BOX EXPOSURE IN 10\*\*003MWD/T  
LOCATION OF MAX. ASS. EXP. IN C/O HEIGHT

Table with columns for assembly number, switch number, integrated exposure, peak exposure, and location. Includes values like 1-05, 2-05, 3-05, 4-05, 5-05, 6-05, 7-05, 8-05, 9-05, 10-05, 11-05, 12-05, 13-05, 14-05, 15-05, 16-05, 17-05, 18-05, 19-05, 20-05, 21-05, 22-05, 23-05, 24-05, 25-05, 26-05, 27-05, 28-05, 29-05, 30-05, 31-05, 32-05, 33-05, 34-05, 35-05, 36-05, 37-05, 38-05, 39-05, 40-05, 41-05, 42-05, 43-05, 44-05, 45-05, 46-05, 47-05, 48-05, 49-05, 50-05, 51-05, 52-05, 53-05, 54-05, 55-05, 56-05, 57-05, 58-05, 59-05, 60-05, 61-05, 62-05, 63-05, 64-05, 65-05, 66-05, 67-05, 68-05, 69-05, 70-05, 71-05, 72-05, 73-05, 74-05, 75-05, 76-05, 77-05, 78-05, 79-05, 80-05, 81-05, 82-05, 83-05, 84-05, 85-05, 86-05, 87-05, 88-05, 89-05, 90-05, 91-05, 92-05, 93-05, 94-05, 95-05, 96-05, 97-05, 98-05, 99-05, 100-05, 101-05, 102-05, 103-05, 104-05, 105-05, 106-05, 107-05, 108-05, 109-05, 110-05, 111-05, 112-05, 113-05, 114-05, 115-05, 116-05, 117-05, 118-05, 119-05, 120-05, 121-05, 122-05, 123-05, 124-05, 125-05, 126-05, 127-05, 128-05, 129-05, 130-05, 131-05, 132-05, 133-05, 134-05, 135-05, 136-05, 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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  
SAN ONOFRE NUCLEAR GENERATING STATION,  
UNIT 3

DOCKET NUMBER (2)  
05000362

YEAR	LER NUMBER (6)	PAGE (3)
85-	0104	04 OF 04

TEXT (If more space is required, use additional NRC Form 3884 (1/77))

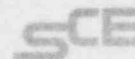
32-000	9.032	9.270	10.254	9.733	10.873	10.386	11.174	10.521	11.144	10.535	10.841	9.944	10.228	9.218	8.945
	11.146	11.423	12.561	12.155	13.334	12.541	13.532	12.798	13.590	12.572	13.264	12.079	12.548	11.283	11.004
	34.000	34.000	34.000	36.000	36.000	36.000	36.000	36.000	36.000	36.000	36.000	36.000	36.000	34.000	34.000
151-05	152-04	153-02	154-04	155-02	156-03	157-31	158-33	159-31	160-33	161-02	162-04	163-02	164-04	165-05	
7.248	3.327	9.278	10.324	10.051	10.861	11.356	11.331	10.311	10.740	9.375	10.225	9.238	8.351	7.221	
8.915	11.014	11.403	12.690	12.259	13.316	12.574	13.392	12.526	13.200	12.130	12.434	11.255	10.923	8.803	
34.000	34.000	35.000	35.000	36.000	36.000	35.000	34.000	36.000	36.000	36.000	38.000	36.000	34.000	34.000	
166-05	167-06	168-04	169-02	170-04	171-02	172-04	173-02	174-04	175-02	176-04	177-02	178-04	179-06	180-05	
5.209	8.832	9.139	9.296	10.348	10.024	10.796	10.107	10.748	9.940	10.270	9.234	9.067	8.750	5.175	
6.431	10.920	11.329	11.388	12.732	12.237	13.268	12.411	13.211	12.136	12.606	11.255	11.133	10.708	6.298	
34.000	35.000	35.000	35.000	36.000	36.000	34.000	34.000	36.000	36.000	35.000	38.000	36.000	34.000	34.000	
181-05	182-02	183-04	184-02	185-04	186-02	187-04	188-02	189-04	190-02	191-04	192-02	193-05			
5.970	7.316	9.032	9.295	10.260	9.301	10.514	9.857	10.139	9.234	9.014	7.435	5.780			
7.514	9.255	11.151	11.387	12.639	12.121	12.968	12.068	12.481	11.305	11.139	9.058	6.998			
34.000	35.000	38.000	36.000	34.000	34.000	36.000	36.000	38.000	38.000	38.000	36.000	34.000			
194-05	195-06	196-04	197-02	198-04	199-02	200-04	201-02	202-04	203-06	204-05					
5.852	8.846	8.984	9.208	10.080	9.602	10.058	9.170	8.939	8.800	5.808					
7.139	10.881	11.087	11.283	12.463	11.735	12.444	11.245	11.030	10.810	7.063					
35.000	36.000	36.000	34.000	36.000	36.000	36.000	36.000	36.000	36.000	36.000					
205-05	206-05	207-07	208-07	209-04	210-07	211-07	212-05	213-05							
5.327	7.221	8.629	10.267	9.143	10.270	8.652	7.201	5.285							
6.534	8.821	10.616	12.659	11.331	12.680	10.591	8.802	6.464							
34.000	34.000	34.000	34.000	34.000	34.000	34.000	34.000	34.000							
214-05	215-05	216-05	217-05												
5.574	7.268	7.269	5.580												
6.805	8.846	8.854	6.826												
34.000	34.000	34.000	34.000												

MAXIMUM INTEGRATED ASSEMBLY EXPOSURE IS 0.112445D+05 MWD/T IN ASSEMBLY 110  
 MAXIMUM PEAK AXIAL EXPOSURE IS 0.137041D+05 MWD/T, OCCURRING AT 36.00 0/0 OF THE CORE HEIGHT IN ASSEMBLY 108  
 CORE AVERAGE EXPOSURE IS 3.923558D+04 MW/T  
 Equal to 243.42 EFPD

----- BATCH AVERAGE EXPOSURES -----

BATCH NUMBER	BATCH NAME	AVERAGE EXPOSURE (GWD/T)
1	A1	10.395
2	A2	9.468
3	B1	11.007
4	B2	9.879
5	C	6.282
6	C+	8.811
7	C+	9.611

*Southern California Edison Company*



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES  
STATION MANAGER

TELEPHONE  
(714) 492-7700

February 27, 1985

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Docket No. 50-362  
30-Day Report  
Licensee Event Report No. 85-004  
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.36(c)(2), 10 CFR 50.73(a)(2)(i), and Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Coolant System specific activity. Neither the health and safety of plant personnel nor the public were affected by this event.

If you require any additional information, please so advise.

Sincerely,  
*JG Haynes*

Enclosure: LER No. 85-004

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)  
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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