

NOV 6 1984

Docket Nos.: 50-338
and 50-339

Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Stewart:

SUBJECT: PROPOSED INADEQUATE CORE COOLING INSTRUMENTATION SYSTEM (ICCIS) -
NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2 (NA-1&2)

We have completed our review of your submittals dated March 7, March 23, July 6 and August 3, 1984, regarding the subject as noted above. Your submittals were in response to our "Request for Additional Information" dated January 16, 1984. The enclosed, Safety Evaluation (SE) incorporates technical input from the staff as well as our technical contractor, the Oak Ridge National Laboratory.

We have determined that: (1) the currently installed Reactor Vessel Level Instrumentation System (RVLIS) will be acceptable upon completion of the full functional test and calibration at the respective startups after the 1984 refueling outages for NA-1&2; (2) the commitments to upgrade the Core Exit Thermocouples (CET) and Subcooling Margin Monitor (SMM), and to update emergency procedures based on the Westinghouse Owners Group (WOG) Emergency Response Guidelines (ERG), Revision 1 are acceptable; (3) the proposed schedule to complete upgrading of the existing CET system prior to or during the 1987 refueling outage and to update EOPs to WOG ERG, Revision 1 in mid-1985 is acceptable; and (4) the proposed technical specification changes to the Accident Monitoring Instrumentation Tables 3.3-10 and 4.3-7 should include the CET system.

Therefore, we find your proposed final ICCIS is in compliance with the Item II.F.2 requirements of NUREG-0737 and is acceptable upon completion of the upgrading of the existing ICCI, the implementation of the revised EOPs, the technical specification changes for the CET system, and the RVLIS functional test and calibration. By this letter and enclosed SE, Multiplant Action Item F-26 is hereby closed for NA-1&2. In addition, this letter and enclosed SE will be used as a guideline by Region II to verify your commitments to

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complete upgrading the existing ICCI (as specified in the enclosed SE) prior to or during the NA-1&2 1987 refueling outages. Final review of the requested technical specifications for the CET system will be handled as plant specific items.

Original signed by:

James R. Miller, Chief
Operating Reactors Branch No. 3
Division of Licensing

Enclosure:
Safety Evaluation

cc: w/enclosure
See next page

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