

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	PAGE (3) 1 OF 0 2
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TITLE (4)
HPCI Inop./ADS Out of Service for Surveillance.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	
10	10	84	84	022	00	11	09	84		
									DOCKET NUMBER(S) 0 5 0 0 0	
									0 5 0 0 0	

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 19 8	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME L.A. Kuczynski - Nuclear Plant Specialist, Level III		AREA CODE 71117	75421-1317519

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		
B	B J	P	B 5 0 8	N							

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 10, 1984, the High Pressure Coolant Injection (HPCI) system was declared inoperable due to high vibration identified during a maintenance check. The HPCI booster pump, main pump, and turbine were realigned and rerun satisfactorily. The remaining Emergency Core Cooling Systems and the Reactor Core Isolation Cooling System were operable throughout this event, except for forty (40) minutes on October 13, 1984. During those forty (40) minutes, the Automatic Depressurization System was inoperable for the performance of a routine surveillance test which had come due. Entry into Limiting Condition for Operation 3.0.3 was duly noted by Operations personnel.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	- 0 2 2	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On October 10, 1984, with the Unit at 98% power, the High Pressure Coolant Injection (HPCI) system was declared inoperable due to high vibration detected during a maintenance check of the HPCI bearings. Limiting Condition for Operation (LCO) 3.5.1 was entered at 2315. Investigation showed that there was misalignment between the booster pump reduction gear unit and the main pump and between the main pump and the turbine. As a precautionary measure, the main pump bearings and the bearings on the pump end of the turbine were inspected. All were found in good condition, cleaned and reinstalled. The booster pump, main pump, and turbine were realigned and the system rerun satisfactorily. LCO 3.5.1 was cleared at 1215 on October 13, 1984.

As part of an approved system modification to replace/upgrade instrument and control components in the HPCI turbine lube oil and control systems, the dowel pins in the turbine pedestal will be upgraded. The reduction gear unit will also be doweled. Both of these actions will control the direction of expansion during heat up and guide both the turbine and reduction gear unit into the hot alignment desired.

The remaining Emergency Core Cooling Systems (ECCS) were operable throughout the event, except for forty (40) minutes on October 13, 1984. During those forty (40) minutes, the Automatic Depressurization System (ADS) was inoperable for the performance of a routine surveillance test of the Residual Heat Removal System - Low Pressure Coolant Injection Mode - Pump Discharge Pressure High (Permissive) pressure switches. Present Station Technical Specifications require the ADS to be declared inoperable during this surveillance. Entry into LCO 3.0.3 was duly noted by Operations personnel. A Technical Specification change will be submitted to the Nuclear Regulatory Commission to provide relief from the present requirement.

PP&L

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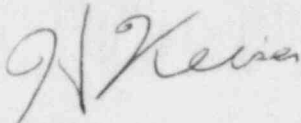
November 9, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-022-00
ER 100450 FILE 841-23
PLAS-007

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 84-022-00. This event was determined reportable per 10CFR50.73(a)(2)(v) and 10CFR50.73(a)(2)(i), in that with the High Pressure Coolant Injection System declared inoperable due to high vibration, the Automatic Depressurization System was removed from service for forty (40) minutes to perform surveillance testing, thus causing entry into Limiting Condition for Operation 3.0.3.



H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

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