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NUCLEAR LICENSING & SAFETY DEPARTMENT

November 2, 1984

U. S. Nuclear Regulatory Commission Office of Nuclear Reactor Regulation Washington, D. C. 20555

Attantion: Mr. Harold R. Denton, Director

Dear Mr. Denton:

SUBJECT: Grand Gulf Nuclear Station

Units 1 and 2

Docket Nos. 50-416 and 50-417

License No. NPF-29 File: 0025/L-860.0

Response to Generic Letter 83-28,

Items 1.1 and 1.2

AECM-84/0465

Generic Letter 83-28 requested conformance to several NRC positions derived from an evaluation of the Salem ATWS events. Attached is the MPGL response to items 1.1 and 1.2 of the generic letter. These items deal with the post trip review program description and procedure (Item 1.1) and data and information capability (Item 1.2). Responses to other items of the generic letter will be provided per the schedule discussed in MP&L letter AECM-83/0723 dated November 4, 1983.

Should you have any questions please feel free to contact us.

Yours truly, SH Hobbs

FOR L. F. Dale Director

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AGR/SHH:rg

Attachments

cc: Mr. J. B. Richard (w/a) Mr. R. B. McGehee (w/o)

Mr. N. S. Reynolds (w/o)

Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a) Office of Inspection & Enforcement U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Mr. J. P. O'Reilly, Regional Administrator (w/a) U.S. Nuclear Regulatory Commission Region II 101 Marietta St., N.W., Suite 2900 Atlanta, Georgia 30323 Member Middle South Utilities System

A055

### Action 1.1 Response

### Post-Trip Review (Program Description and Procedure)

The program for review and analysis of unscheduled reactor trips at Grand Gulf Nuclear Station (GGNS) is described in an administrative procedure 01-S-06-26, Post Trip Analysis. The Post Trip Analysis procedure is a comprehensive procedure whose purpose is to provide guidance for the evaluation of unscheduled reactor trips to verify that the plant safety-related trip systems functioned as intended and that the plant may be restarted safely. It covers responsibilities of the plant staff and describes the various analyses required to determine the cause of the trip and ability to restart. The Post Trip Analysis procedure references other procedures including a scram recovery procedure, 03-1-01-4, which are to be used in recovering from a scram.

### Item 1.1.1

The basic restart criteria for GGNS is defined in the On-Shift Analysis portion of procedure O1-S-06-26. The On-Shift Analysis is performed by the Shift Technical Advisor (STA) with assistance from the Shift Supervisor as needed with the purpose of assessing the plant response, degrmining the cause of the trip and verifying that safety-related systems respond as expected. Upon achieving controlled plant conditions, the STA will begin the On-Shift Analysis utilizing available data to determine the following:

- (1) If all major safety-related and other important equipment involved in the trip operated as expected.
- (2) If the transient caused any detrimental effects on plant equipment.
- (3) If a Technical Specification Safety Limit was exceeded.
- (4) If it is acceptable to restart the reactor.

Upon completion of the On-Shift Analysis, the STA and Shift Supervisor must classify the event into one of four categories in order to determine the process for restart. These four categories are:

- Class I The cause of the trip is positively known and has been corrected; all safety-related and other important equipment functioned properly during the trip.
- Class II The cause of the trip is positively known and has been corrected: some safety-related equipment did not function properly; however, the malfunction has been corrected or a Technical Specification constraint does not prohibit a startup.
- Class III The cause of the trip is not positively known.
- Class IV Some safety-related and/or other important equipment functioned in an abnormal or degraded manner during the trip and the malfunction has not been corrected or prevents startup due to Technical Specification constraints.

The On-Shift Analysis determination is then reviewed by the on-call Operations Superintendent and the on-call Technical Support Superintendent to determine the thoroughness, accuracy and consistency of the trip analysis. Also, the

Shift Superintendent must notify the Duty Manager of the results of the analysis.

The process used to determine when to restart the reactor is linked to the On-Shift Analysis classification of the event. For Class I or Class II events the decision to restart is made by the GGNS General or Duty Manager. The decision to restart after a Class IV event is made by the General Manager after necessary corrective actions have been taken. Class III events require determination of a course of action leading to plant restart which will be made by the Plant Safety Review Committee (PSRC) or the Safety Review Committee (SRC). The recommended course of action for Class III events should include:

(1) The actual or most probable cause of the trip.

(2) The maintenance and testing recessary before reactor restart including additional measures to verify the most probable cause.

(3) Additional monitoring or trending required during and/or after reactor restart.

(4) Necessary briefings to Operations and/or Maintenance personnel concerning specific equipment indications or possible malfunctions.

(5) Conditions necessary for reactor restart.

The decision to restart after a Class III event is made by the General Manager after the PSRC recommended actions have been addressed.

### Item 1.1.2

The review and analysis of the unscheduled reactor trip will be performed by the STA with assistance from the Shift Supervisor. Input to the review process comes from individuals involved in the event including operators and maintenance personnel. The responsibilities and authorities of all persons involved in the post trip analysis and their role in making the decision to restart the reactor are defined in the Post Trip Analysis procedure. These responsibilities are summarized as follows:

GGNS General Manager - The General Manager or Duty Manager is responsible for ensuring that an adequate analysis of unscheduled reactor trips is conducted and for authorizing plant restart after an unscheduled reactor trip.

Manager, Plant Operations - The Manager, Plant Operations is responsible for reviewing the results of analyses of the transient events.

Technical Superintendent - The Technical Superintendent is responsible for designating personnel under his supervision to perform the in-depth analysis of transient events and for evaluating the results of analyses of transient events.

Operations Superintendent - The Operations Superintendent is responsible for providing in-depth support to the Technical Section as required to complete the analyses.

Shift Superintendent - The Shift Superintendent is responsible for:

Ensuring that the On-Shift Analysis required by the procedure is performed.

Reporting the results of the On-Shift Analyses to the GGNS General Manager/Duty Manager prior to obtaining permission to restart the reactor.

Shift Supervisor - The Shift Supervisor is responsible for assisting the Shift Technical Advisor in the performance of the On-Shift Analysis.

Shift Technical Advisor - The Shift Technical Advisor (STA) is responsible for:

Collecting the data required to perform the Post Trip Analyses.

Performing the On-Shift Analysis.

Reporting the results of the Post Trip Analysis to the Shift Superintendent.

Transmitting the Post Trip Analyses and available data to the Technical Superintendent for the followup analysis.

Ensuring that the GETARS is operating in "SENTINEL MODE" whenever the reactor is in Operational Condition 1 or 2 and startup testing requiring use of GETARS is not in progress.

Additional responsibilities of the STA and Shift Supervisor are defined in GGNS FSAR Section 13.1.

### Item 1.1.3

The qualifications and training for responsible personnel are described in the GGNS FSAR sections 13.1 and 13.2. Job requirements and training for these positions are in accordance with the requirements of ANSI Standard N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel."

The Shift Supervisor's qualification requirements are discussed in FSAR Section 13.1.2.2.3.2 and include the requirement that he hold an SRO license. The STA's position is to contribute to maximizing plant safety during transient or accident situations by independently assessing plant conditions and by providing the technical assistance necessary to mitigate the incident and minimize the effect on personnel, the environment, and plant equipment. STA qualifications and training are described in FSAR Section 13.2.1.2.10 and includes training on plant systems, administrative controls and general operating procedures, transient and accident analysis and emergency procedures, and simulator training. The STA also attends an instructional period devoted to accessing and interpreting information supplied from the process computer.

### Item 1.1.4

The sources of plant information necessary to conduct the post trip review and analysis are discussed in Attachment I, Item 1, of procedure 01-S-06-26 (see Attachment 3 to this letter) and include data from the BOP, NSSS, and GETARS

Sentinel Computer Systems, plant recorders and strip charts, statements from personnel involved in the transient, and copies of shift logs and turnover sheets. Specific information related to parameters monitored and information recorded by this equipment is provided in our response to section 1.2 of Generic Letter 83-28. Also, General Electric (GE) has conducted a study (at the request of the Salem ATWS Generic Issues Committee of the BWROG) on the adequacy of typical BWR control room instrumentation and has concluded that control room instrumentation is adequate to determine initiation of a safety system at the required setpoint. There is sufficient on/off indication to verify proper system flow path and that initiation has gone to completion. GE also studied eight typical BWR transients to determine if available control room instrumentation (i.e., Panel Alarms, Recorders, Computer Systems) could be used to determine the cause of the scram. The results of their review showed that in general the cause could be determined with available instrumentation, augmented by operator and technician investigation when necessary.

### Item 1.1.5

Procedure 01-S-06-26 requires the STA to use the available plant data to determine if all major safety-related and other important equipment involved in the trip operated as expected and if any Technical Specification Safety Limit was exceeded. The On-Shift Analysis portion of the procedure specifically requires the STA to verify:

- (1) That the RPS operated properly by verifying that the first tripped parameter in each RPS channel caused the RPS channel to trip prior to manual scram.
- (2) That the limiting safety system settings in the RPS Evaluation Data Table (see Attachment 3) were not exceeded without an RPS trip signal.
- (3) If any of the ECCS or RCIC systems initiated.
- (4) If SRV's opened, a turbine trip occurred or a Recirc Flow Control Valve runback occurred.
- (5) If a Recirc pump trip or Recirc pump transfer to LFMG set occurred.

Additionally, the STA will determine if there was any abnormal radiological response or a malfunction or trip of any other components or systems.

It is the position of MP&L, that if it is determined that a particular system should have initiated for a particular event then it need only be established that the system did indeed initiate and that initiation was in the proper sequence. A detailed analysis of the actual performance of that system following an unscheduled shutdown is not a criterion for restart. Such an analysis is accomplished through the normal surveillance testing procedure done at regular intervals to verify system operability. This step is consistent with the philosophy espoused in Caution No. 1 of the NRC approved BWR EPGs. Confidence in the accuracy of control room readout is provided both by the routine maintenance and surveillance activities performed on plant systems, and by normally scheduled and performed calibration activities. Adherence to these efforts precludes the need to enter into a complete recalibration (i.e., pressure, flow, operating times, etc.) or performance reevaluation of the adequacy of system operation prior to restart from an unscheduled trip.

### Item 1.1.6

As discussed in Item 1.1.1, procedure 01-S-06-26 requires the STA and Shift Supervisor to classify the transient into one of four categories in order to determine the process for restart. For a Class III event (cause of trip not positively known) a determination of a course of action leading to plant restart must be made by the PSRC or SRC using any or all expertise available. The PSRC must provide a recommended course of action for Class III events to the General Manager which includes:

- (1) The actual or most probable cause of the trip.
- (2) The maintenance and testing necessary before reactor restart including additional measures to verify the most probable cause.
- (3) Additional monitoring or trending required during and/or after reactor restart.
- (4) Necessary briefings to Operations and/or Maintenance personnel concerning specific equipment indications or possible malfunctions.
- (5) The conditions necessary for reactor restart.

The decision to restart the plant after a Class III event is made by the General Manager after PSRC recommended actions have been addressed.

In addition to these requirements, the procedure requires an Off-Shift Analysis to be performed by one or more persons in the Technical Section. Persons performing the Off-Shift Analysis will have an SRO license or be a Certified STA or have equivalent experience. The purpose of the Off-Shift Analysis is to verify the method and results of the On-Shift Analysis, summarize investigative and corrective actions, initiate further corrective actions as required, formulate recommendations to prevent reoccurrence and develop a report on the event.

Guidelines for preservation of physical evidence to support an independent analysis of the transient are provided in the Post Trip Analysis procedure. The guidelines call for the STA to

- (1) Mark chart recordings with date, time, recorder number, parameter, and time scale/chart speed to aid in interpretation of data.
- (2) Transfer the GETARS Sentinel Program data from disc to magnetic tape in order to obtain hard copy plots and allow retention of data.
- (3) Collect handwritten statements from individuals involved in the event (to include plant conditions, individual actions, plant response, noted equipment malfunctions and procedural deficiencies).
- (4) Collect data to complete Attachment I of 01-S-06-26 (see Attachment 3 of this letter) which includes pre-trip plant conditions, the Sequence of Events Log from the BOP and NSSS Alarm Typer, the Post Trip Log from the BOP and NSSS Alarm Typer, plots of several GETARS parameters and copies of shift logs and turnover sheets.

### Item 1.1.7

As stated in the introduction to Item 1.1, the program for review and analysis of unscheduled reactor shutdowns is contained in GGNS administrative procedure 01-S-06-26, Post Trip Analysis. This comprehensive procedure references several other procedures including Scram Recovery, 03-1-01-4, and Cold Shutdown to Generator Carrying Minimum Load, 03-1-01-1, which are to be utilized in recovery from an unscheduled reactor trip. Copies of these GGNS procedures are maintained on-site and are available for NRC staff review as required. X81rg5

### Action 1.2 Response

### Post-Trip Review (Data and Information Capability)

The Shift Technical Advisor is responsible for collecting the data required to perform the post trip analysis. The information collected should be sufficient to reconstruct the transient. Attachment 3 to this letter lists the information to be collected by the STA for use in determining the cause of an unscheduled reactor trip. The sources of this information include the NSSS Post Trip Log and Sequence of Events Log, the BOP Post Trip Log and Sequence of Events Log, the GETARS Sentinel Program, control room panel instrumentation, shift logs and turnover sheets, recorder and strip chart output, and written statements from individuals involved in the event. The objective of the STA's On-Shift Analysis is to determine the condition of the unit prior to the event, if all major safety-related and other important equipment involved in the trip operated as expected and if the transient caused any detrimental effects on plant equipment. The STA is to be alert for abnormal indications or degraded trends in equipment performance, events occurring out of the normal or expected sequence, and unexpected alarms. The following information is provided in response to Generic Letter 83-28 Section 1.2 action request.

### NSSS Computer

The NSSS Computer for GGNS is a Honeywell HS-4000 computer with a Printronix printer. This computer monitors sequence of events as well as time history variables related to NSSS functions. The Sequence of Events (SOE) Log provides digital on/off indication. Points are scanned on the order of milliseconds and a printout of all SOE points that alarm (in the sequence it alarms) is given when any SOE point alarms. The Post Trip Log provides analog indication. Points are scanned once every five seconds and are printed out every thirty seconds. Data is recorded and printed for the four minute period prior and four minute period after a trip signal is generated. A hard copy of the SOE and Post Trip Logs is printed out on the alarm printer in the Control Room. The NSSS Computer and multiplexers are powered by the Class IE, uninterruptible power source.

The parameters that are monitored by the NSSS computer SOE Log are:

\*Reactor High Pressure (CH A-D) \*Vessel Low Water Level (CH A-D) \*Drywell High Pressure (CH A-D) \*Manual Scram Div 1 or 3, 2 or 4 \*Reactor Scram Div 1 or 3, 2 or 4 RHR/ADS DW Pressure (CH B, F) RHR/ADS Water Level (CH B, F) LPCS/RHR/ADS DW Pressure (CH A, E) LPCS/RHR/ADS Water Level (CH A, E) \*TSV Closure (CH A-D) \*TCV Fast Closure (CH A-D) HPCS Drywell Pressure (CH C, L, C, R) HPCS Low Water Level (CH C, L, G, R) Recirc Pump Trip (CH A, B) Turbine Bypass Valve RHR Pump Breaker (Loop A, B, C)

Relief Valve Pilot Solenoid (No 1-20) \*Main Steam Line High Radiation (CH A-D) RHR Pressure (Loop A, B, C) LPCS Pump Breaker RHR Injection Flow (Loop A, B, C) LPCS System Pressure LPCS Low Flow Bypass Valve HPCS Pump Breaker No. 2 HPCS Pressure HPCS Low Flow Bypass Valve IRM Upscale Level (CH A-H) \*Main Steam Line Isolation (CH A-D) Vessel High Water Level (CH A-D) APRM Neutron Flux (CH A-H) \*Discharge Volume Water Level High (CH A-D) \*Neutron Monitoring System (CH A-D) APRM Thermal Power (CH A-H)

The format for displaying these parameters gives the computer point number, the parameter name and the status of the alarm (i.e. on, off, reset).

The Reactor Frotection System alarm points are marked with an asterisk on the SOE Log listed. An optical isolator and one or more relays is associated with each point. In order for the point to be printed on the alarm printer, two sources of power must be available. These sources are the plant batteries (125 VDC) and the Motor Generator sets (MG, 125 VAC).

The parameters monitored by the NSSS Post Trip Log are:

APRM Flux Level (CH A)
Reactor (RTR) Feedwater Flow (Loop A, B)
Steam Dome Pressure
Reactor Water Level
Total Steam Flow
RTR Feedwater Inlet Temp (CH A)
RTR Core Pressure Drop
RTR Total Core Flow
Main Turbine Steam Flow

The format for this log prints out the computer point number and the value of the parameter every 30 seconds.

#### BOP Computer

The BOP Computer for GGNS is a SEL-32/55 computer. This computer monitors sequence of events as well as time history variables related to BOP functions. The SOE Log provides digital on/off indication and points are scanned on the order of milliseconds. A printout of all SOE points that alarm (in the sequence it alarms) is given when any SOE point alarms. The Post Trip Log provides analog indication. Points are scanned once every five seconds and are printed every thirty seconds. Data is recorded and printed for the four minute period before and four minute period after a trip signal is generated. A hardcopy of the SOE and Post Trip Logs is printed out on the alarm printer in the Control Room backed up by an on demand BOP printer in the Control Room. The BOP Computer and multiplexers are powered by a Class IE, uninterruptible power source.

The parameters displayed for the SOE Log include information and status on the following major components/systems:

Recirc Pump Motor Reactor (water level) Offgas Post Treatment (radiation) Suppression Pool Makeup Valves (and Water Level) Division 1 or 2 Drywell Purge Drywell/Containment Differential Pressure Division 1 or 2 LOCA Trip Vent Isolation Containment Isolation or Auxiliary Isolation Condensate Pumps Reactor Feed Pump Turbine (RFPT) Heater Drain Pumps Feedwater Heaters Heater Drain Tank Turbine Condenser Vacuum Main Generator Transformers Diesel Generators Loss of Offsite Power Sources SGTS Division 1 or 2

The format for printing these variables gives the computer point number, a description of the parameter, and a set (status) message. A complete listing of all parameters monitored is given in Table I.

The parameters monitored by the Post Trip Log include information for the following systems/variables:

Recirculation System
Reactor Water Level
APRM Flux
Turbine/Condenser
Drywell Pressure
Reactor Feed Pump Turbine/Reactor Feed Pump
Condensate Pumps & Booster Pumps & Demineralizers
Feedwater Heaters
Moisture Separator & Reheaters
Main Generator

The format for this log gives computer point number, parameter description, high/low sensor values and a status message. A complete listing of all parameters monitored is given in Table II.

#### GETARS Sentinel Program

The hardware for the GETARS Sentinel Program is a Hewlett Packard model 2117-F computer with strip chart recorder output. In the Sentinel mode, 234 channels are monitored. The maximum sample speed is 100 samples per second per channel. When one of the selected (50 channels maximum) channel signals is determined to be outside of its prescribed limit, the limit checking mode is terminated and the Sentinel mode begins. Records are maintained from thirty seconds

preceding the trip to approximately ten minutes afterwards. All data is recorded on hard disc and is then transcribed onto a magnetic tape for permanent storage. In addition, all data may be output to a time-distance strip-chart. The power source for this computer and multiplexers is class IE, uninterruptible power. The pertinent parameters monitored in \*he Sentinel mode as a minimum include the following:

Recirc Loop A Temperature Recirc Loop B Temperature Narrow Range Level Narrow Range Dome Pressure Wide Range Dome Pressure Total Steamline Flow Condenser Vacuum APRM B Total Core Flow APRM A APRM C Wide Range Level APRM D RC1C Pump Flow HPCS Flow "A" RHR System Flow LPCS Flow "A" HP Control Valve Position

### Control Room Panel Instrumentation

Table III lists some of the Control Room instrumentation which is available to monitor operation during and following a transient. The first column gives the system name and parameter to be monitored, the second column states whether the parameter is recorded on a strip chart or recorder, and the third column indicates if the parameter is recorded on the operations round sheets. The systems listed were selected in the GE study as important systems to monitor. Additional recorder and strip chart information which is available and of interest in the post trip review is listed in Table 1 of Attachment 3 to this letter.

#### Summary

In addition to the specific information discussed above, other sources of information for assessing the cause of a Reactor trip exist including written statements from persons involved in the event (operators, maintenance personnel) and shift logs and turnover sheets. As to the adequacy of available equipment, MP&L believes that the results of the GE study are valid for GGNS and, therefore, GGNS Control Room instrumentation is adequate to verify safety system operation as needed for the post trip analysis.

POINT ID COMPUTER DESCRIPTION	TYPE	FREQ LOSENSR HISENSR	LOENG HIENG EUNIZS SET/LLIM	K21/HPIU	MESSGE ILEVAN
POINT ID COMPUTER DESCRIPTION					
N19H002B CONDENSATE BOOSTER PUMP B	SOE	0015		NO TRIP	
NI9MOOZC CONDENSATE BOOSTER PUMP C	SOE	0015		NO TRIP	
N19MOO3A CNDS PMP A HTR PROT DEVICE	SOE	0018	OPERATE		ALARM-S
N19M0038 CNDS PMP B MTR PROT DEVICE	SOE	0015	OPERATE	RESET	ALARM-S
N19H003C CNDS PMP C MTR PROT DEVICE	SOE	0015	OPERATE		ALARM-S
N19H004A CONDENSATE PUMP A	SOE	0015	TRIPPED	NO TRIP	
		0015	TRIPPED	NO TRIP	ALARM-S
N19M004B CONDENSATE PUMP B N19M004C CONDENSATE PUMP C N19M003A CNDS PMP A SUCT PRESSURE N19M003B CNDS PMP B SUCT PRESSURE	SOE	0015	TRIPPED		AUARM-S
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M19N003C CNDS PMP C SUCT PRESSURE M19N053A CBP A SUCT PRESS	SOE	0015	I/OM	NOT LOW	ALARM'S
MIGNOSTA CRP & SUCT PRESS	SOE	0015	POM	NOT LOW	ALARM-S
		0015	LOW	NOT LOW	ALARM-S
N19N053C CBP C SUCT PRESS	SDE	0015	PUM	NOT LOW	ALARM-S
N19N105 CNDS PMPS HOTWELL WTR LEVEL	SOE	0015	I'UM-TOM	NOT LOLO	
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M21J612B RFPT B TRIP VLV M21J612B RFPT B TRIP VLV M21K005B RFPT A RECIRC FLOW M21K005B RFPT B RECIRC FLOW	SOF	0015	OPERATE		ALARM-S
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MZIKOOSA REPT A RECIRC FLOW	SOF	0018	FOM	NOT LOW	ALARM-S
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#21#208A RFPT A BRG PRESSURE SW PS8	aue		LOW	NOT LOW	
M21M208B RFPT B BRG PRESSURE SW PS8	SUE	0018	FOM	NOT LOW	
M21M209A RFP A BRG PRESSURE SW PS9	SOE	0015	LOW	NOT LOW	
NZIN2098 RFP B BRG PRESSURE SW PS9	SUE	0015	HIGH	NOT HIGH	
N21N212A REPT A INACT THE BEG WE PS12	SOE	0015	HIGH	NOT HIGH	
M21M212B RFPT B INACT THR BRG WR PS12	SUE	0015	TRIPPED		
WZIWZIGA REPT A TRIP FLUID PRESS PSIG	205	0018	TRIPPED	RESET	ALARM-S
#21#2160 RFPT B TRIP FLUID PRESS PS16	SOE	0018	FOM	NOT LOW	
	SOE	0015	LON	NOT LOW	
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N23M002B HTR DR PMP B EMERG TRIP	SUE	0015	ALARM	RESET	ALARM-S
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	SUE	0018			NO TOTAL	
	SOE	0018			MO TRIP	
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DE DE LA	SOE	0018		NEW		
SED LEVEL	SOE	0018		IN-IH		
	SOE	0018		H-HI		
а.	SUE	0018		HI-HI		
B DRAIN	SOE	0018		IH-IH	NOT HIH!	
GROUND	SOE	0018		TRIPPED	RESET	ALARM-S
GEN LOSS FLO T	SOE	0018		ALARM	RESET	ALREN
GEN	SOE	0018		ALARM	RESET	VIANA O
GEN EXCESS V/H	SOE	0018		ALARM	RESET	ALAKA
The part of the last		3 2 5 4				

POINT ID	COMPUTER DESCRIPTION			*******	*********	*******	 ********		
M415707	LOW FREG GND FAULT TRIP	SOE	0015				ALARM	RESET	ALARM-S
N415708	MAIN GEN DIFF RLY TRIP	SOE	0015	-			ALARM	RESET	ALARM-S
M415709	UNIT DIFF RLY TRIP	SOE	0015				ALARM	RESET	ALARM"S
N415710	MAIN GEN NEG PH SEQ TRIP	SOE	0015				ALARM	RESET	ALARM-S
H418711	MAIN GEN DISTANCE RLY TRIP	SOE	0015				ALARM	RESET	ALARM-S
N415712	MAIN GEN REV PWR RLY G12 TR	SOE	0015				ALARM	RESET	ALARM=S
N415713	UNIT ON LINE	SOE	0018				IN OPER	NOT OPER	
N415714	PT 11 FAILURE	SOE	0015				RESET	ALARM	ALARM-R
N415715	MAIN GEN LO RLY G11 TRIP	SUE	0015				RESET	ALARM	ALARM-R
	MAIN GEN LU RLY G12 TRIP	SUE	0015				RESET	ALARM	ALARM-R
	MAIN GEN GHD OC RLY TRIP	SOE	0015				ALARM	RESET	ALARM-S
	MAIN GEN LO RLY G13 TRIP	SOE	0015				RESET	ALARM	ALARM-R
	MAIN GEN LO RLY G14 TRIP	SDE	0015				RESET	ALARM	ALARM-R
	MAIN GEN REV PWR RLY UTIL TR	SOE	0015				 ALARM	RESET	ALARM=S
	MAIN GEN LO RLY UT11 TRIP	SOE	0015				RESET	ALAh.1	ALARM-R
	PT 12 FAILURE	SOE	0015				RESET	ALARM	ALARA-R
	PT 13 FAILURE	SOE	0015	-			 RESET	ALARM	ALARM-R
	MAIN XFMR DIFF RLY TRIP	SOE	0015				ALARM	KESET	ALARM-S
	MAIN XFMR NEU GROUND RELAY	SOE	0012				TRIPPED	RESET	ALARMOS
	MAIN XFMR A SUDDEN PRESSURE		0015		-		 TRIPPED	RESET	ALARM-S
	MAIN XFMR B SUDDEN PRESSURE	SUE	0015				TRIPPED	RESET	AT.ARM-S
CONTRACTOR OF THE PARTY OF THE	MAIN XFMR C SUDDEN PRESSURE	100000000000000000000000000000000000000	0015				TRIPPED	RESET	ALARM-S
	BRKR J5228 FAILURE TRIP	SOE	0015				ALARM	RESET	ALARN-S
	BRKR J5232 FAILURE TRIP	SOE	0015				ALARM	RESET	ALARM-S
	GEN FIELD BRKR OPEN TRIP	SOE	0015				ALARM	RESET	ALARM-S
	MAIN GEN SKR AUTO TRIP J5228		0015			فيتنفيذ المستددات	ALARM	RESET	ALARM-S
	MAIN GEN BER AUTO TRIP J5232		0015				A!/ARM	RESET	ALARM-S
	MAIN GEN PILOT EXTR SH CKT	SOE	0015				ALARM	RESET	ALARM-S
	MAIN GEN ROTUR GND TRIP	SUE	0015	-			 ALARM	RESET	ALARM-S
	PW LEAKAGE TRIP	SOE	0015				ALARM	RESET	ALARM-S
CONTRACTOR OF THE PROPERTY.	PW STAT CIRCUIT LOW FLOW	SOF	0015				FOR	NOT LOW	ALARM-S
100-100-741-100-1000-1001-1001-1001-1001	PW ROT CIRCUIT LOW FLOW	SOE	0015				 POM	NOT LOW	ALARK-S
	PW PH A CIRCUIT LOW FLOW	SOE	0015				I'OA	NOT LOW	ALARM-S
	PW PH B CIRCUIT LOW FLOW	SOE	0015				I'OM	NOT LOW	ALARM-S
		SUE	0015	_			LOW	NOT LOW	ALARM-S
	PW PH C CIRCUIT LOW FLOW	SOE	0015				TRIPPED	RESET	ALARM-S
#43K229	PW TANK LEVEL	SOE	0015				TRIPPED	RESET	ALARM-S
	GEN TERM BOX LEVEL						Charles and the second	The state of the s	
	E. C. SECTE CONTRACTOR CO.	SOE	0015				HIGH	NOT HIGH	
		SOE	0015				HIGH	NOT HIGH	
	Market II a Market I a market	SOE	0015				LOM	NOT LOW	ALARM-S
	SJAE B STEAM FLOW	SUE	0015				LOW	NOT LOW	ALARM-S
	OFF-GAS DISCH TO VENT VALVE		0015				CLOSED	NOT CLSD	
		SOE	0015				TROUBLE	NORMAL	ALARM-S
	CIRC WIR PHP B MUTOR PROT	SOE	0015				TROUBLE	NORMAL	ALARM-S
	CIRC WIR PMP A EMERG TRIP	SOE	0015				ALARM	RESET	ALARM-S
		SOE	0015				ALARM	RESET	ALARM-S
		SOE	0015				DOM	NOT LOW	
		SOE	0015				FOM	NOT LOW	ALARM-S
*******	DIV 1 STANDBY SERV WATER SYS	SOE	0015				IN OPER	NOT OPER	ALARMOS.

	DIESEL GENERATOR A					RUNNING	STOPPED	ALARM-S
PROFOIS	DIESEL GENERATOR B	SOEL	0015				STOPPED	ALARN-S
1000010	SPARE SEQ EVENT CONTACT CHAN					SET	RESET	HONE
1004030E	SPARE SEQ EVENT CONTACT CHAN	SUE	0015			SET	RESET	HONE
02525EL	SPIRE SEQ EVENT LOGIC INPUT	SOEL	0015	and the second second second	 	SET		HONE
0253SEL	SPARE SEQ EVENT LOGIC INPUT	SOEL	0015			SET	RESET	HONE
002545EL	SPARE SEG EVENT LOGIC INPUT	SOEL	0015			SET	RESET	HONE
00255SEL	SPARE SEG EVENT LOGIC INPUT	SOEL	0015			SET	RESET	MONE ALARM-S
8155701	SERV XFMR 11 PRIMARY LO RLY	SOE	0015			OPERATE		ALARM-S
9155702	SERV XFMR 11 BU LO RLY	SOE	0012			OPERATE	RESET	ALARM=S
P15s711	SERV XFMR 11 SUDDEN PRESS RY	SOE	0018			OPERATE		ALARM-S
R158725	SERV XFMR 11 PRI DIFF RELAY	SUE	0015			OPERATE		ALARM-S
R155726	SERV XFMR 11 OVERCURRENT RLY	SOE	0015			OPERATE		ALARM-S
R155727	SERV XFMR 11 NEUTRAL D/C RLY	SOE	0015			OPERATE		ALARM-S
R155728	SERV XFMR 11 BU DIFF RELAY	SOE	0015			OPERATE		ALARM-S
R15S730	GRNDG XFMR 11 O/C TRIP RELAY	SOE	0015			OPERATE		ALARM-S
R155731	GRND XFMR 11 NEU O/C TRIP	SOE	0015			OPERATE		ALARM-S
R155732	GRNDG XFMR 11 OV TRIP RELAY	SUE	0015			OPERATE		ALARM-S
R155733	GRNDG XFMR 21 O/C TRIP RELAY	SUL	0015		 	OPERATE		ALARM-S
R155734	GRNDG XFMR 21 NEU O/C TRIP	SUE	0015			OPERATE		ALARM-S
R155735	GRNDG XFMR 21 DV TRIP	SUE	0015			OPERATE		ALARM-S
R158737	SERV XFMR 21 PRI L/O TRIP SERV XFMR 21 SEC L/O TRIP	SUE	0015		 	OPERATE	RESET	ALARM-S
R158738	SERV IFAR 21 SEC L/U IRIP	SOF	0015			OPERATE	RESET	ALARM-S
R155740	SERV XFMR 21 SUD PRESS TRIP	SOF	0015			OPERATE	RESET	ALARM-S
R158753	SERV XFMR 21 PRI DIFF TRIP SERV XFMR 21 O/C RELAY TRIP	SOF	0015			OPERATE	RESET	ALARM-S
H155754	SERV XFMR 21 NEUTRAL O/C RLY	SOF	0015			OPERATE	RESET	ALARM-S
R130/33	SERV AFRE 21 BU DIFF RELAY	SOE	0015			OPERATE	RESET	ALARM-S
K129/20	BUS 15AA UNDERVOLTAGE RELAT	SOEL	0015			OPERATE	The state of the s	ALARN-S
0216738	BUS 16AB UNDERVOLTAGE	SOEL	0015			TRIPPED		ALARM-6
#216755	LOSS OF OFFSITE PWR SOURCES	SOEL	0015			ALARM	RESET	ALARM-S
2715761	LOSS OF OFFSITE PWR SOURCES	SOEL	0015			ALARM	RESET	ALARMAS
74816144	SGTS DIVISION 1	SOE	0015				NOT OPER	
T48L6148	SGTS DIVISION 2	SOE	0015			IN OPER	NOT OPER	ALARM-S
	234 RECORDS PRINTED							

NSSSA NSSSA NSSSA NSSSA LSA LSA LSA NSSSA	0055 0055					MS/H MS/H			
NSSSA NSSSA LSA LSA LSA					-	ME/H	The state of the s		The state of the s
NSSSA LSA LSA LSA									
LSA LSA LSA						M#/H			
LSA LSA LSA						Me/H	all Carling	ALESS BILL	A STATE OF THE PARTY OF THE PAR
LSA		32MV	160MV	0		IN	33.7		MNYY
LSA		32MV	160MV	0		IN	33.7		ничт
	0058	32MV	160MV	0	60	IN	33.7	39.7	нихх
			- The State of the	-		PSI	TO THE SALES	MESTA	A Children on the Control of the Control
LSA		32MV	160MV	0			20		MAAA
LSA	0058	32MV	160MV	0	100		20	80	ANYY
NSSSA	AND DESCRIPTION OF					*PWR			Charles and the second second second
NSSSA						&PWR			
NSSSA						&PWK			
NSSSA	Walter Street, St. Co.	THE RESERVE		-	THE REAL PROPERTY.	%PWR	TE STILL SE		<b>建设的现在分词 医克拉克氏 医克拉克氏 医</b>
NSSSA						#PWR			
NSSSA						*PWR			
NSSSA		-7-2		THE PARTY OF		*PWR	A THE REST		<b>自然を持ちませた。</b>
NSSSA						APWR			THE PART OF THE PA
LSA	0058	1 V	5 V	0					HNNY
LSA	0055	11	54	0					HNNY
LSA	0058	14	5 V	0					HNNY
LSA	0055	14	5 V	0		I The second sec	The second secon		MNNY
LSA	0055	14	57	0					HNNY
LSA	0055	14	SV	0		The second secon			MNNY
LSA	0058		5 V	0	The second secon				HNNY
LSA	0055	14	177.00	0					HHHY
LSA	0058	2000					CONTRACTOR A PROPERTY OF THE PARTY OF THE PA		HHNY
LSA	0055	14		0					MNY
LSA	0055					T. (1)			MMNY
LSA	0058							45	HNNY
LSA	0058				The second second second				
LSA	0058		The second secon						
LSA	0105	14					T. T		
LSA	0105	17							
LSA	0105					PSIG	20		[ [ [ [ [ [ [ [ [ [ [ [ [ [ [ [ [ [ [
LSA	0108			The second second		P210	20		нихх
LSA	0105	14		0				The second second	HMAN
LSA	0105	17		0	0.000	The same of the sa		The second secon	MNYN
LSA	0105			0					UNAA
LSA	0105			0					HNYY
LSA	0108		104	0				V	HNYY
LSA	0058								MNYY
FOW	0055								HNYY
LSA	0055	17							HNYY
LSA	0105	-							MAA
LSA	0105			0					HHYY
LSA	0108			0			The state of the s		
LSA	0015	- 20		0		The state of the s	150		HNYT
LSA	0018			0					MNYN
	NSSSA NSSSA NSSSA NSSSA NSSSA LSA LSA LSA LSA LSA LSA LSA LSA LSA	NSSSA NSSSA NSSSA NSSSA NSSSA NSSSA LSA 005S LSA 010S LSA	NSSSA NSSSA NSSSA NSSSA NSSSA LSA 005S LSA 005S LV LSA 010S	NSSSA NSSSA NSSSA NSSSA NSSSA  LSA 005S 1V 5V LSA 010S 1V 5V	NSSSA NSSSA NSSSA NSSSA NSSSA LSA 005S 1V 5V 0 LSA 005S 1V	NSSSA NSSSA NSSSA NSSSA NSSSA NSSSA NSSSA LSA 005S 1V 5V 0 100 LSA 005S 1V 5V 0 100 LSA 005S 1V 5V 0 100 LSA 005S 1V 5V 0 200 LSA 005S 1V 5V 0 1250 LSA 010S 1V 5V 0 1250 LSA 010S 1V 5V 0 1250 LSA 010S 1V 5V 0 30 LSA 010S 1V 5V 0 5 5.5 LSA 005S 1V 5V 0 30 LSA 010S 1V 5V 0 300 LSA 010S 1V 5V 0 300 LSA 010S 1V 5V 0 300 LSA 001S 1V 5V 0 300	NSSSA LSA OOSS 1V 5V 0 100 PSIG LSA OOSS 1V 5V 0 200 PSIG LSA OOSS 1V 5V 0 1250 PSIG LSA OOSS 1V 5V 0 30 IN HGV LSA OOSS 1V 5V 0 30 IN HGV LSA OOSS 1V 5V 0 30 IN HGV LSA OOSS 1V 5V 0 30 IN LSA OOSS 1V 5V 0 30 OPSIG CSA OOSS 1V 5V 0	NSSSA NSPWR NSSSA NSSSA NSPWR NSSSA NSSSA NSPWR NSSSA NSSSA NSPWR NSWR NSSSA NSSA NSSSA NS	NSSSA NSSA NSSSA NSSA NSSSA NS

**	CBP SU	SUCT HDR	PRESS	LSA	0013	11	SV		160	PSIG	35	150	11/25/10
850N			-	LSA	0015	A .		0.0	000	MLB/HR	•		AANN
190801	CRP NE	RECIRC FL	TE COMIN TEN POSM	LSA	0018		28	00	9009	PSIG	360	570	
140	SP HTR	LP HTR CON DUTL	TL FLOW	LSA	0058	11	AS	0	13	MLB/HR			
#19H120	COND H	HWL MKUP	- 143	LSA	0055	24	104	0	100		•	100	HNYY
M122	COND H	HWE REJ	7.7	LSA	0055	24	107	0	100		•	100	HNYY
419N166	LP HTR	LP HTR OUTL FLO		LSA	0055	14	SV	0		MLS			NNN
11	CNDS P	MPS REC	CNDS PMPS RECIRC CONT OUT	LSA	0018	14	24	0	100		0.	100	700
198004	HOTWEL	HOTWELL LEVEL CONT	CONT OUT	LSA	0055	14	SV	0	100		0	100	
1198630	CBP RE	CIRC FL	ON CONT OUT	LSA	0018	17	84	0	100		0	100	THE STATE
4214037A	RFP A	DISCH PRESS	RESS	LSA	0105	14	24	0	1500	PSIG	1000	1350	701
21N037B	m	DISCH PRES	RESS	LSA	0108	14	AS	0	1500	PSIG	1000	1350	700
218076A	RFP A	SUCT PR	PRESS	LSA	0018	14	24	0	200	PSIG	133	200	
121N078B	0		103	LSA	0018	14	24	0	200	PSIG	135	200	70.0
21NOB4A	4	RECIRC	VLV POSM	LSA	0018	3 /	107	0	100		0	100	
21N0848	RFP B	RECIRC	WLW POSM	LSA	0018	24	104	0	100			100	30
087A	-		FLOW	LSA	0018	14	24	0	=	MLB/HR			N.N.
421N087B	10	SUCTION	FLOW	LSA	0015	14	24	•	*	MUB/HR			LINNN
21NOSBA	4	SUCTION	-	LSA	0015	1 A	SV	0		MLB/HR			HINN
21N088B	1 00	SUCTION		LSA	0018	14	54	0	*	MLB/HR			9
21N1141	*	LPSY P	OSW	LSA	0105	11	SV	0	100		0	100	*
2181148	100	-	POSN	LSA	0105	11	54	0	100		0	100	HHYY
5.8	RFPT A		POSN	LSA	0108	11	SV	0	100		0	100	E
W21N115B			POSN	LSA	0105	11	AS	0	100		0	100	HNYY
4.4		FEED	# D	LSA	0055	14	AS	0	8000	RPM	1200	2000	-
121N4648	REACTO		PUMP B SPEED	LSA	0055	14	20	0	0009	RPM	1200	2000	E
21R003A	RFP A		CONT	LSA	0018	10	SV	0	100		0	100	*
2130038		œ		LSA	0018	14		0	100		0	100	
122N100			ens.	LSA	0055	3.6	100	0	003		0	100	2
122H151		_	DIFF PRESS	LSA	0005	A .	20	0	100	PSID		#	MININ
22R603	10	HIN	DIFF CONT	LSA	0055	11	28	0	100		0	100	THE
123NOO3A		7	73/37	LSA	0055	1.5	20	0	0	IN MC	1.25	•	HILL
123N003B		48	LEVEL	LSA	0058	1.4	20	0	12	IN MC	1,25		THE
ZINGOREZ		29		LSA	0055	)A	24	0	-	IN MC	1.25	•	-
123N013A	FOW HIR	R 4A DR	NEW.	LSA	0058	34	104	0			•	100	7
123N013B		48	AFA	LSA	0058	3.4	100	0	100		0	100	
123N013C	E	24	VLV POSN	LSA	0055	24	104	0	0	100	0	100	
123M018A	FD# HTR	3.8	LEVEL	LSA	0058	3.6	24	0		IN MC	2,25	•	HINKE
423N018B	FD# HTR	38	LEVEL	LSA	0035	**	SV	0	15	IN MC	2.25	•	HAK
123MOISC	FOW HIR	36	TEAST	LSA	0055	11	24	0	-	IN MC	2.25	¥	HATT
#23H027A	a	R 3A DR	VALVE POSITION	LSA	0055	24	104	0	0		•	100	HNYY
823M027B		A 38 DR	VALVE	LSA	0055	24	104	•	100		0	100	HNYY
123N027C	П	3C DR	VALVE	LSA	0055	24	104	0	6		0	100	HNYY
#23N030A	FDW HTR	2.8	LEVEL	LSA	0055	11	20	0		IN MC	2,125	•	HAY
123N030B	FOW HTR	28	TEVEL	LSA	0055	14	80	0	15	IN MC	2,125	4	HNYY
123H030C		35		LSA	0055	4	254	0.	-	IN MC	1,125		HARL
#23N036A		33	-	LSA	0108	4:	200	0 0	900	KLS/HK	000	575	MAN
123N0368	FOW HTR	R 28 DRAIN	AIN FLOW	LSA	0105	14	24	0	000	KLB/HR	90	575	HHII

										•	***	AAnn	
-	HTR 2A	R VALVE	LSA	0058	37	104	0	100			100	1100	
	HTR 28			8500	**	104		000			100	MANU	
	HTR 2C	745	_	5000		200		200	KI B / ND		***	MANA	
	HTR IA	DENIN FLUS	1.81	0100		200	0	120	KER/KR		110	HNYY	The second second
N23N050B F	25		1.54	010		28	. 0	120	KLB/HR		110	HNYY	
	HTR 64	200	LSA	0058	14	AS.	0	15		•	9	HHEE	
1 40	HTR 68	LEVEL	LSA	0055	3.6	54	0	15	IN NC	0		TANK	
	HTR 6A	DRAIN FLOW	LSA	0108	1 8	28	0	2.5	ML8/HR	0.3	2.4	MAK	
	HTR 68	FLOW	LSA	0108	11	SV	0	*	MLB/HR	0.3	2.4	HNY	
	HTR 6A	VALVE	LSA	0055	20 1	104	0 1	001		00	001	ILNE	
0	HTR 68		LSA	0055	24	104	0 (	001	-		001	AANN	
	DR TX	- 1	LSA	0000	1	40		000	IN SC		0,	MANA	
	DR		LSA	0108	*	200		•••	MED AND		3.4	HNYY	
0	DR PMP	SCH	123	2010	***			100	ALD VIEW		1001	MAKE	
-	DR TR	APA	LOA	0000	4.7	100		001		0	100	MARK	-
00	DR TK C	POS	104	5000	***	101	•	000	Detc	280	570	HNYY	
	E :	UISCH HOR PREDS	407	0000		**	. 0	100			100	HNYY	
	MIN GA MUNN	1800		2000		200		100		0	100	HNYY	-
	9		403	0000		200		100		. 0	100	HNYY	
	*	THE COME OUT	40.	0000	. >	**	. 0	100		0	100	HNYY	
	DEN SA NORM	THUS.	NO.	0000		200	0	100		0	100	HNYY	
E 000700000	MIN JO MUNN	CONT	LAB	0058	3.6	AS		100		•	100	HINT	
	28	CONT	LSA	0055	14	SV	•	100		0	100	HNYY	
Œ	28	CONT	LSA	25:00	11	37	0	100	-	•	100	HANA	
	20	V CONT	LSA	0055	14	AS.	0	100		0	100	HNYY	
	6.8	CONT	LSA	0055	11	34	0	100		0	100	HNY	
	6A NO	V CONT DUT	LSA	0055	4	45		100		00	100	2000	
ī	æ	WLV CONT	LSA	0058		40		000		•	000		
	ST BEA	REA	LSA	0108	11	AC	0	200			190	200	
	CONT		LSA	0018	4 1 1	ZORA		000		00	001	NEVA	
=	CONT	~		0018	-	KOZ		200			000	******	
	CONT	(4	LSA	0018	4 11 4	ZOMA	0	100			100	HANN	
0	DMT	POSM	158	0018	4 M M	ZONA		000			001	HAND	
	RUST BRG	CONT		0010		2000		200		•		MAGA	
N30M035A L	4	5	100	2000		*****		000					
308035B L	10	3	407	0000		2000		100			000	AARN	
N30N035C L		3	203	0000	***	2000		1001			200	MMAN	
M SONOTH F	•	2 2	100	00000	***	2088		100			100	*****	
T BOSONOSA	TURE B NO.	CA POSE	LEA	0100	***	ZONA		100			100	NHYY	
		APC	LSA	0108	11	5.4	0	300			190	MNKN	
	TUDB DE	BOT L	LSA	0108	17	AS	0	300	-	-	190	MANA	-
	3 4 CD 5 C		LSA	0018	4MA		0	2100	RPH	1710	1850	HNYY	
		SOUTH VLV POSN	LSA	0108	4 H.A	20MA	0	100				HNYY	
	SPR	POSK	LSA	0105	488	ZOMA	0	100	-	0	100	HHYY	10000
	ITG STH	VLV POSM	LSA	0108	AO.	100	•	100		•	100	MMEY	
	CHAIL DES			Athe	A S	**	•	***		<	400	MWVV	
	2 1 2 2 2		603	0010	**			224		,	***	Umas.	

POINT ID	COMPUTER DESCRIPTION		and the second second second second	AND THE RESIDENCE OF THE PARTY	HISENSR	********			*******		******	************
							25	IN WC		17.5	HNYV	
		Läk	0058	14	5V	0		IN WC		17.5		
	Underen Sant Sin to a morrow	LSA	0055	14	SV	0				15.5		
	CONTRACTOR MATERIAL MATERIAL CONTRACTOR CONT	LSA	0058	14	5 V	0		IN WC	5	15.5	C Control Control	
		LSA	0055	17	5V	0		IN WC	,	1.4		
		LSA	0105	14	54	0		MLB/HR			HHYN	
35M009B		LSA	0105	17	5V	0		MLB/HR		Part of the Control o	нети	
35N010A	reading was well as to	LSA	0055	2 V	104	0	100		0	100		
35N010B	means were but so w	LSA	0055	24	107	0		3		15.5		
35N012A	A WAR STORES WAS AND AS TO MAKE THE	LSA	0058	14	5 V	0		IN WC		15.5	The state of the s	
	* *** ** *** *** *** ** ** ***	LSA	0055	14	5 V	0		IN WC			HALM	
358020A	1 STG RHR DR TK A DR FLOW	LSA	0105	14	5 V	0		KLB/HR			HNAM	
	* 614 11-11 511 411 4 611 1-401	LSA	0108	14	54	0		KLB/HR		The second second	The state of the s	
358021A	* 6*8 201101 811 821 10 811 10 10 1	LSA	0055	2 V	104	0	100		0	The state of the s	HNAA	AUDIO GENERALES
35N0213	1 STG RHR DR TK B DR VLV	LSA	0055	24	104	0		*	0		HNYY	
35N024A	2 STG RIR DR TK & LEVEL	LSA	0055	14	5 V	0		IN WCV			HNYY	
35N024B		LSA	0055	14	5 V	0		IN MCA			HMXM	
		LSA	0105	14	54	0		KL8/HR		-	HNYN	
35N031B	2STG RHR DR TK B DR FLOW	LSA	0105	14	5 V	0		KLB/HR			HNYY	
	2STG RHR DR TK A DR VLV POSN	LSA	0058	24	104	0	100	The state of the s	0			
35N032B	ZSTG RHR DR TK B DR YLV POSN	LSA	0055	2 V	104	0	100		0		HMYY	
35N038A	SEP SHL DR TK A DR VLV POSN	LSA	0055	2 V	10V	0	100		0		HMYY	
35N038B	SEP SHL OR TK B DR VLV POSN	LSA	0055	24	104	0	100	The second second second	0		HNYY	
	MSTRE SEP SHELL DR TK FLOW	LSA	0055	14	5 V	0		"I.B/HR		1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	HNYN	Figure 1 State of the State of
35N0418	MSTRE SEP SHELL DR TK FLOW	LSA	0055	17	5 V	0		KLB/HR		1	HNAM	
	MSR DR TK NORM VLV CONT OUT	LSA	0055	14	5 V	0	100		0		HNYY	
135R041B	MSR DR TK NORM VLV CONT OUT	LSA	0055	17	5 V	0	100		0		HMAA	
358043A	ISTG DR TK NORM VLY CONT OUT	LSA	0055	17	5 V	0	100	The second secon	0		HNYY	
35R043B	1STG DR TK NURM VLV CONT OUT	LSA	0055	17	5V	0	100	The second second	0		HNYY	
358045A	2STG DR TK NORM YLV CONT OUT	LSA	0055	17	54	0	100		0		нихх	NEW YORK TO SERVE THE REAL PROPERTY.
3580458	2STG DR TK NORM VLV CONT OUT	LSA	0058	17	5 V	0	100		0		HNYY	
3520565		LSA	0058	14	5 V	0	100		0		HNYY	
3580568	SEP SHL DR TK B NORM CONT	LSA	0055	17	57	0	100		0		HMAA	PRODUCTION OF THE PARTY OF THE
41K601		LSA	0015	4HA	20MA	0	1400	HW		1372.5		
	WTR CONN HDR VIB RESULTANT 2	LSA	0055	4MA	20MA	0	30	MILS			HNYN	
		LSA	0105	17	54	-1000	1000	MYAR	w330		HMAA	NATIONAL PROPERTY.
	MAIN GEN WATTS	LSA	0015	OV	4.5V	0	1400	MW	280	1372.5		
43N240	PW STATOR CIRCUIT LOW FLOW	LSA	0105	4HA	20MA	0	725	GPM	555	700	HNYY	
	PW ROTOR CIRCUIT LOW FLOW	LSA	0105	4MA	20MA	0	725	GPM	614	700	HHAA	
439242	PW PHASE A CIRCUIT LOW FLOW	LSA	0105	4MA	20MA	0	30	GPM	24	28	HMAA	
439242		LSA	0105	4MA	20MA	0	30	GPM	24	28	NNYY	
439244	PW PHASE C CIRCUIT LOW FLOW		0105	4HA	20MA	0	30	GPM	24	28	HHYY	
428244	SJAE A STEAM FLOW	LSA	0105	14	5 V	0	13.5	KLS/HR	3	11	HNYY	
62M013M	SJAE B STEAM FLOW	LSA	0105	14	5 V	0	13.5	KLB/HR	3	11	HNYY	
CAMPIER.		LSA	0105	17	57	0	994	LB/HR	200	800	HNYY-	
1625015A		LSA	0105	14	54	0	994	LB/HR	200	800	HNYY	
10480130	RADIAL WELL PMP COM DISCH	LSAS	0025	17	5 V	0	150	PSIG	60	115	HALAA	
4/MUU3	SUPPLY HEADER PRESSURE	LSAS	0105	19-	57	0		PSI	95	125	HETT	
PRHORE	INSTR. AIR DISCH HOR PRESS	LSA	0108	17	5 V			PSIG	95	130	HMZY	

Table III
CONTROL ROOM PANEL METER INDICATIONS AVAILABLE

System/Parameter	Control Room Indication	Recorded on Round Sheet
RCIC System		
Flow	Yes	No
Main Steam Pressure to the Turbine	Yes	No
Turbine Speed	Yes	No
Turbine Exhaust Pressure	Yes	No
Pump Suction Pressure	Yes	No
Pump Discharge Pressure	Yes	No
RCIC Controller	Yes	No
HPCS System		
Pump Discharge Pressure	Yes	No
Pump Flow	Yes	No
Selected Valve Positions	Yes	No
Recirc System		
A/B Recirc Pump Differential Pressure	Yes	Yes
A/B Loop Flow	Yes	Yes
A/B Pump Speed	Yes	Yes
A/B Pump Motor Amperage	Yes	Yes
Recirc Flow Controllers	Yes	Yes
Feedwater System		
RFP Suction	Yes	No
RFP Discharge Pressure	Yes	Yes
Turbine Control Valve Position	Yes	No

### Table III

System/Parameter	Control Room Indication	Recorded on Round Sheet
RFP Turbine Speed	Yes	No
Feedwater System Controllers	Yes	No
RHR/LPCS System		
RHR Pump A, B and C Amps	Yes	No
Service Water Flow to RHR Heat Exchangers	Yes	No
RHR Pump A, B, and C Flow	Yes	No
RHR Hx A, B Level	Yes	No
RHR Hx's to RCIC Pressure	Yes	No
RHR Hx's A, B Pressure	Yes	No
RHR A Head Spray Flow	Yes	No
Selected Valve Positions	Yes	No
Hx A, B Outlet Conductivity	Yes	No
RHR Controllers	Yes	No
LPCS System		
LPCS Pump Amps	Yes	No
LPCS System Flow	Yes	No
ADS		
SRV Tailpipe Temperature Recorder	Yes	No
DG (1, 2 and 3)		
Fuel Day Tank Level	Yes	Yes
Fuel Storage Tank Level	Yes	Yes
DG Field Amps	Yes	No
DG Field Volts	Yes	No
DG Vars	Yes	No
V81re11		

### Table III

System/Parameter	Control Room Indication	Recorded on Round Sheet
DG Watts	Yes	No
DG rpm	Yes	No
DG ø Amps	Yes	No
DG A-B & Volts	Yes	No
Div Battery Volts	Yes	Yes
Emergency Bus Volts	Yes	Yes
Incoming Bus Volts	Yes	No
Running Bus Volts	Yes	No
Synchroscope	Yes	No

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GGNS UNIT
POST TRIP ANALYSIS
\*-SCRAM NO.

TIME/DATE OF INITIATING EVENT	
Routine Maintenance, Power	
Changes, System Lineups,	
etc.)	number from Reference 3.7.
PLANT CONDITIONS PRIOR	
Reactor Thermal Power	DATA SOURCE
Total Core Flow	
RPV Water Level	
RPV Pressure	psig/psia
Generator Output	
Steam Flow	
FW Flow	
Offgas & Radwaste BLDG. Vent Activity	MR/HR
Mode Switch Position	
Flow Control Mode	
Neutron-Flux Monitored by: APRM IRM Specify IRM Ranges if Known A	SRM B

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## ON-SHIFT ANALYSIS

1.		ain the data lis kage.	sted below. If unavailable, explain. A	Attach to this data ttached (Yes/No)
	a)	Sequence of Eve	ents log from BOP Alarm Typer	
	b)	BOP Post Trip	Log from BOP on-demand Typer	
	c)	Sequence of eve	ents log from NSSS Alarm Typer	
	d)	NSSS Post Trip	Log from NSSS Alarm Typer	
	e)	Recent copies	of 0D-3, 0D-7 and P1 (0D-17)	
	f)		ollowing parameters as needed ntinel Program:	
		Channel	Parameter	Included yes/no
		15	A Recirc Loop Temperature	
		16	B Recirc Loop Temperature	
		66	Narrow Range Level	
		67	Narrow Range Dome Pressure	
		68	Wide Range Dome Pressure	
		73	Total Steamline Flow	
		107	Condenser Vacuum	
		112	APRM B	
		166	Total Core Flow	
		170	Vessel Bottom Head Drain Temp	
		174	APRM A	
		215	APRM C	
		223	Wide Range Level	
		384	APRM D	
		197	RCIC Pump Flow	
		219	HPCS Flow	
		248	"A" RHR System Flow	
		256	LPCS Flow	
		287	"A" HP Control Valve Position	

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# ON-SHIFT ANALYSIS (CON'T)

	g) :	Copies of shift logs, turnover sheets, and appropriate chart recordings (See Attachment 3, Table 1 for a list of recorders of possible interest.)
	h)	Written statements from individuals involved in the event. (Reactor operator, technician, Shift Supervisor, etc.)
	i)	A copy of the Incident Report generated
	j)	A Copy of the NSSS I/O typer output
	k)	Any other information considered important (specify)
2.		m the NSSS sequence of events log, fill in following information:
	RPS	Channel A trip time
	Par	ameter causing trip time
	RPS	Channel B trip time
	Par	ameter causing trip time
	Tim	me of manual scram
3.	in	rify that RPS operated properly by verifying that the first tripped parameter each RPS channel caused the RPS channel to trip prior to manual scram. (N/A manual scram was initiating event). Verified:  STA OR SS initials
4.	lin	tempt to verify using the GETARS PLOTS and installed recorder charts that the miting safety system settings listed in Attachment III, Table 2 were not seeded without an RPS trip signal. Verified:  STA or SS initials.
	If	unable to verify, explain why.

### ADMINISTRATIVE PROCEDURE

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## ON-SHIFT ANALYSIS (CON'T)

- 5. Ensure that the GETARS Sentinel program data, if obtained, is transferred from disc to mag tape. Label the tape with date, time and scram number.
- 6. Did an isolation occur: Yes/No If yes, then list type, cause, and time:
- 7. Did any of the following ECCS systems initiate: Yes/No

### Actuation signal Time Operation sat?

HPCS LPCI A LPCI B LPCI C

- 8. Did RCIC initiate: Yes/No If yes, then list cause and time and steady state flow rate.
- Did one or more SRV's open? Yes/No If yes, list valve number(s) and time of opening. List data sources used for determination.
- 10. Did a turbine trip occur: Yes/No Cause Time
- 11. Did a Recirc FCV runback occur? Yes/No Cause Time
- 12. Did a Recirc pump trip occur: Yes/No Cause Time
- 13. Did an automatic Recirc Pump transfer to the LFMG occur? Yes/No Cause Time
- 14. Was there any abnormal radiological response? List any abnormal change in area radiation monitoring, process radiation monitoring, and environmental radiation monitoring indications.

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ON-SHIFT ANALYSIS
(CON'T)

15. What is the probable cause of the Reactor trip?

16. List any components or systems for which a malfunction or trip was observed and the time if known.

17. Fill in the indicated data for the transient:

MAX	MIN	DATA SOURCE
	N/A	
	MAX	==

Verify that the Technical Specifications Safety Limits were not exceeded.

STA or SS

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## ON-SHIFT ANALYSIS (CON'T)

- 18. Classify the event into one of the following categories:
  - Class I The cause of the trip is positively known and has been corrected; all Safety-Related and other important equipment functioned properly during the trip.
  - Class II The cause of the trip is positively known and has been corrected: some Safety-Related equipment did not function properly; however, the malfunction has been corrected or a Technical Specification constraint does not prohibit a startup.
  - Class III The cause of the trip is not positively known.
  - Class IV Some Safety-Related and/or other important equipment functioned in an abnormal or degraded manner during the trip and the malfunction has not been corrected or prevents startup due to Technical Specification constraints.

		Performed by SRI	O or STA/ D	ate / Time		
		Concurrence Sh:	ift Superinte	ndent / Date	/ Time	
19.	Duty manager	notified Name		/Date / Tim	e Noti	ier
	Comments					
	The State of the Land of					

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### ON-SHIFT ANALYSIS (CON'T)

POST TRIP SUMMARY (Describe the sequence of significant events leading to the transient up to achieving post-transient control of the plant. Discuss any equipment operating problems or anomalies, as well as any unusual operator action taken to mitigate the consequences of the transient.

### NOTE

It is permissible to attach copies of the appropriate Shift Log Book entries or additional sheets as necessary.)

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### SAMPLE FORMAT

Post Trip Analysis Report

Scram	Number		Date/Time	
-------	--------	--	-----------	--

#### I. SUMMARY

- A. Narrative of the Event
- B. Statement of the cause of the reactor scram (automatic or manual)
- C. Statement of adequate safety system performance
- D. Chronological tabulation of events
- II. Verification of On-Shift Analysis
  - A. Statement of Concurrence
  - B. Discrepancies noted in On-Shift Analysis method and results
- II. Summary of Corrective Action
  - A. Results of investigative action
  - B. Results of corrective action
- IV. Observations/Recommendations

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## Table 1

## RECORDERS AND STRIP CHARTS

## OF INTEREST

PARAMETER	PANEL	INSTRUMENT NO.
IRM/APRM	1H13-P680-5B	1C51-NR-R603A (CH E-RED PEN) (CH A-BLUE PEN)
IRM/APRM	1H13-P680-5B	1C51-NR-R603C (CH G-RED PEN) (CH C-BLUE PEN)
IRM/APRM	1H13-P680-78	1C51-NR-R603B (CH F-RED PEN) (CH B-BLUE PEN)
IRM/APRM	1H13-P680-7D	1C51-NR-R603D (CH H-RED PEN) (CH D-BLUE PEN)
RECIRC LOOP SUCTION TEMP	1H13-P614	B33-R604
VESSEL, HEAD FLANGE & DRAIN LINE TEMPS	1H13-P614	B21-TR-R643
RADWASTE VENT. EXH. RAD. LEVEL	1H13-P600	D17-R600 (RED PEN -CPM)
OFFGAS PRETREATMENT. RAD MONITOR	1H13-P600	U17-R604(CPM)
OFFGAS POST-TREATMENT RAD MONITOR	1H13-P600	D17-R601 BLACK-TRAIN A (MR/HR) RED-TRAIN B (MR/HR)
DRYWELL FIREUS	1H13-P870-3B	1M71-PDR-R601A (RED PEN)
DRYWELL PRESS	1H13-P870-9B	1M71-PDR-R601B (RED PEN)
DOME PRESS (WIDE RANGE)	1H13-P601-20B	B21-UR-R623A (BLUE PEN)

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## RECORDERS AND STRIP CHARTS

## OF INTEREST

PARAMETER	PANEL	INSTRUMENT NO.
VESSEL LEVEL (WIDE RANGE)	1H13-P601-17B	B21-UR-R623B (RED PEN)
DOME PRESS (NARROW RANGE)	1H13-P680-9B	C34-R609
VESSEL LEVEL (UPSET RANGE)	1H13-P680-2B	C34-UR-R615 (GREEN PEN)
VESSEL LEVEL (WIDE RANGE)	1H13-P680-2B	C34-UR-R615 (BLUE PEN)

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Table 2

RPS EVALUATION DATA

PARAMETER	TRIP STRT	ALLOWABLE VALUE
TANADETEN	That Siris	TALUE
TRM High	( 120/125	< 122/125
Inn, nigh	- 120/123	- 122/123
ADDM Nout non		
	/ 45W 0TD	200 070
Linx urdu	- 13% KIP	< 20% RTP
ADDM Nout non	/ 1100 DTD	< 120% RTP
	110 KOIT 2	- 120% KIP
LION BIVELD	/ //	2 22 54
	Max C III's KIP	Max 113% KIF
riux nign		
DDV Process	1064 7	4070 7
Wish	_ 1064.7 psig	- 10/9.7 psi
uldu		STREET, STREET
DDV 1 1		
	< 55.5 In	< 54.1 in
High		
00V 11		
	2 11.4 in	> 10.8 in
LOW		
MSIV-Closure	≤ 6% Closed	< 7% Closed
WC 11		
	≤ 3.0 x FPB	< 3.6 x FPB
High		
Drywell Press	≤ 1.23 psig	< 1.43 psig
High		•
		The Royal District
	≤ 60%	< 63%
	≥ 40 psig	> 37 psig
	> 44.3 psig	> 42 psig
The same of the sa		
Reactor Mode		Marine Marine
Switch -		
Shutdown	N/A	N/A
Man SCRAM		
CHA & C		
		THE RESERVE AND ADDRESS OF THE PARTY OF THE
	RPV Level, High  RPV Level, Low  MSIV-Closure  MS Line Radiation, High  Drywell Press High  SDV Level, High  TSV-Closure (Low Oil Press) TCVFC- (Low Oil Press) Reactor Mode Switch - Shutdown Man SCRAM	IRM, High < 120/125  APRM Neutron Flux High < 118% RTP  APRM Neutron Flux High

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Table 3

NSSS ISOLATION DATA

BOP COMPUTER POINT	PARAMETER	NOMINAL TRIP STPT.	ALLOWABLE TRIP STPT.
		INMENT ISOLATI	
*#			
M71L604A/B	- DIV 1 & 2	ISOLATION	
N/A	RPV Level,	> -41.6 in	> -43.8 in
N/A	Drywell Press., High	> 1.23 psig	≤ 1.43 psig
D17L642A,B	Vent Exh. Rad., High	< 2.0 mr/hr	< 4.0 mr/hr
B21L606	MANUAL	N/A	N/A
	SECONDARY CON	TAINMENT ISOLA	TION
*# M71L606A/B	- DIV 1 & 2	AUX ISOLATION	
N/A	RPV LEVEL,	> -41.6 in	> -43.8 in
N/A	Drywell Press., High	≤ 1.23 psig	≤ 1.43 psig
D17L611 #	Fuel Handling Exh. Rad., Area Vent High - High	< 2.0 mR/Hr	< 4.0 mR/Hr
D17L646A,B	Fuel Handling Area Pool	≤ 18 mR/Hr	< 35 mR/Hr
B21L606	MANUAL	N/A	N/A

Unless otherwise denoted by a "#" all points are also available from the NSSS Computer under an equivalent NSSS Computer Point Number.

\*SOE Point

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## NSSSS ISOLATION RESPONSE EVALUATION

BOP COMPUTER	DADAMETER	NOMINAL	ALLOWABLE
POINT	MAIN STEAM	TRIP STPT.	TRIP STPT.
+4			
B21J100A-D-	MSL ISOLA	TION CHA THRU	D,
N/A	RPV Level 1	≥ -150.3 in	≥ -152.5 in
D17L633A,B	MS Line Radiation High	≤ 3.0 x FPB	< 3.6 x FPB
B21L603	MS Line Pressure, Low	< 849 psig	< 837 psig
E31N686A thru E31N686D	MS Line Flow, High	≤ 169 psid	≤ 176.5 psid
N/A	Condenser Vacuum	> 9 in. Hg Vac	≥ 8.9 in. Hg Vac
E31L619A	MS Line Tunnel Temp, High	< 185°F	< 191°F
E31L620A	MS Line Tunnel DT, High	< 101°F	< 104°F
B21L606	MANUAL	N/A	N/A

Unless otherwise denoted by a "#" all points are also available from the NSSS Computer under an equivalent NSSS Computer Point Number.

\*SOE Point

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## NSSSS ISOLATION RESPONSE EVALUATION

BOP COMPUTER POINT	PARAMETER	NOMINAL TRIP STPT.	ALLOWABLE TRIP STPT
	RWCL	ISOLATION	
E31L614	Differential Flow, High	> 79 gpm	<u>&lt;</u> 89 gpm
E31L634	Differential Flow, Timer	< 45 Sec	≤ 57 sec
E31L616A	Hx Room Temp, High	< 120°F	< 126°F
E31L616B,C	Temp, High	≤ 170°F	< 176°F
E31L616D	Vlv Nest Room Temp, High	≤ 135°F	< 141°F
E31L616E,F	RWCU Demin Room Temp, High	≤ 139°F	< 141°F
E31L616H	RWCU Tank Room. Temp, High	< 139°F	< 145°F
E31L620B	Equipment Area DT High	Var.	Var.
E31L616G	RWCU Demin Valve Room, Temp, High	< 135°F	< 141°F
N/A	RPV Level,	≥ -41.6 in	≥ -43.8 in
E31L619A	MS Line Tunnel Temp, High	<u>&gt;</u> 185°F	< 191°F
E31L620A	MSL Tunnel DT, High	< 101°F	< 104°F
N/A	SLCS Initiation	N/A	N/A
B21L606	MANUAL	N/A	N/A

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# Table 3 (Continued) RCIC ISOLATION DATA

BOP COMPUTER POINT	PARAMETER	NOMINAL TRIP STPT.	ALLOWABLE TRIP STPT
E31L601A,B	RCIC Steam Line DP, High	< 363" H <sub>2</sub> 0	< 371" H <sub>2</sub> 0
E31L600A,B	Low	≥ 60 psig	≥ 53 psig
E51L610A,B	Press, High	≤ 10 psig	< 20 psig
N/A	RCIC Equip Room Temp, *	< 185°F	< 191°F
N/A	RCIC Equip Room DT, High	≤ 125°F	< 128°F
E31L619A	MSL Tunnel Temp, High	< 185°F	< 191°F
E31L620A	MSL Tunnel DT, High	< 101°F	< 104°F
N/A	MSL Tunnel Temp, Timer	≤ 30 min	< 30 min
E31L619C	RHR Equip Room Temp, High	< 165°F	< 171°F
E31L620C	RHR Equip Room DT, High	< 99°F	< 102°F
N/A	RHR/RCIC Steam Flow, High	< 145" H <sub>2</sub> 0	≤ 160" H <sub>2</sub> 0
N/A	MANUAL	N/A	N/A
B21N69's A,C,B,F,	Drywell Press., High	< 1.39 psig	≤ 1.44 psig

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## ECCS ACTUATIONS & INTERLOCKS DATA

BOP COMPUTER POINT	PARAMETER	NOMINAL TRIP STPT.	ALLOWABLE TRIP STPT.
	LPC I/LF	CS INITIATION	
E12L611A#	RHR SYSTEM 1	ACTIVATED	
E12L611B#	RHR SYSTEM 2	ACTIVATED	
E12L605#	RHR HT EXCH	CLG WATER IN TH	EMP
B21N691 A,E,B,F	RPV Level,	> -150.3 in.	> -152.5 in.
B21N694 A,E,B,F	Drywell Press., High	< 1.39 psig	≤ 1.44 psig
N/A	Start Time Delay	< 5 sec	< 5.25 sec
N/A	MANUAL	N/A	N/A
	HPCS IN	ITIATION	
B21N673 C,L,G,R	RPV Level,	> -41.6 in.	≥ -43.8 in.
B21N667 C,L,G,R	Drywell Press, High	≤ 1.39 psiq	≤ 1.44 psig
E22L509	MANUAL Initiation	N/A	N/A
C34L604*# B21-N683A thru B21N683B	RPV Level,	< 53.5 in.	< 55.7 in.
E22L600	CST Level,	> 0 inches	> -3 inches
E22L603	Suppression Pool Level, High	< 5.9 in.	< 6.5 in.

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## ADS INITIATION

BOP COMPUTER POINT	PARAMETER	NOMINAL TRIP STPT.	ALLOWABLE TRIP STPT.
B21L610 A,E,B,F #	- ADS LOGIC		T THE STITE
B21N680A thru B21N680D	RPV Level, Low (Permissive)	≥ 11.4 in.	≥ 10.8 in.
B21N691 A,E,B,F	RPV Level,	≥ 150.3 in.	≥ 152.5 in.
B21N694 A,E,B,F	Drywell Pressure, High	≤ 1.39 psig	< 1.44 psig
N/A	ADS Timer	≤ 105 sec	< 117 sec
E21N653	LPCS Discharge Press, High (Permissive)	145 psig (Increasing)	125-165 psic
E12N656A thru E12N6456C	LPCI Cisch. Press High (Permissive)	125 psig (Increasing)	(A) 115-135 (B/C)115 psig (Increasing)
B21L615A,B	MANUAL	N/A	N/A

<sup>\*</sup> SOE Point

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### ATWS TRIPS

BOP COMPUTER POINT	PARAMETER	NOMINAL TRIP STPT.	ALLOWABLE TRIP STPT.
B33L609A B33L609B	RPT A RPT B	N/A	N/A
N/A	RPV Level,	> -41.6 in.	> -43.8 in.
N/A	RPV Pressure	< 1125 psia	≤ 1140 psig
	END-UF-CYCLE	RPI (IU LEMG)	
C71N605A thru C71N605D	TCV-Closure (Low Oil Pressure)	> 40 psig	<u>&gt;</u> 37 psig
C71N606A thru C71N606D	TSV-Closure (Low Oil Pressure)	> 44.3 psig	> 42 psiq

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## RCIC ACTUATIONS AND INTERLOCKS

BOP COMPUTER POINT	PARAMETER	NOMINAL TRIP STPT.	ALLOWABLE TRIP STPT.
N/A	RPV Level,	≥ -41.6 in.	≥ -43.8 in.
E51L618	MANUAL INITIATION	N/A	N/A
B21N683A thru B21N683D #*C34L604	RPV Level, High	≤ 53.5 in.	< 55.7 in.
E22L600#	CST Level,	≥ 0 in.	≥ -3 in.
E22L603 #	Suppression Pool Level, High	< 5.9 in.	< 6.5 in.

<sup>\*</sup> SOE Point

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### SAFETY RELIEF VALVE DATA

## RELIEF FUNCTION

BOP COMPUTER POINT	VALVE	NOMINAL SETPOINT (psig)	ALLOWABLE RANGE (psig)	
B21F051D	F051D	1103		
B21F047A	F047A 1113		1101/1125	
B21F047C	F047C	1113	1101/1125	
B21F047D	047D F047D 1113		1101/1125	
B21F047G F047G		1113	1101/1125	
B21F047H			1101/1125	
B21F047L	F047L	1113	1101/1125	
B21F051A	F051A	1113	1101/1125	
B21F051B	F051B	1113	1101/1125	
B21F051F	F051F	1113	1101/1125	
B21F051K	F051K	1113	1101/1125	
B21F041A	F041A	1123	1111/1135	
B21F041B	F041B	1123	1111/1135	
B21F041C	F041C	1123	1111/1135	
B21F041D	F041D	1123	1111/1135	
B21F041E	F041E	1123	1111/1135	
B21F041F	F041F	1123	1111/1135	
B21F041G	F041G	1123	1111/1135	
B21F041K	F041K	1123	1111/1135	
B21F051C	F051C	1123	1111/1135	

Unless otherwise denoted by a "#" all points are also available from the NSSS Computer under an equivalent NSSS Computer Point Number.

## SAFETY FUNCTION

VALVE	NOMINAL SETPOINT (psig)	ALLOWABLE RANGE (psig)	
F041A,41B, 41C,41D,41E, 41F,41G,41K	1165	1153/1177	
604/A,47C, 47D,47G,47H 47L	1180	1168/1192	
F051A,51B, 51C,51D,51F, 51K	1190	1178/1202	

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## SAFETY RELIEF VALVE RESPONSE EVALUATION

## LOW-LOW SET FUNCTION

BOP COMPUTER POINT	VALVE	NOMINAL OPEN STPT. (psig)	ALLOWABLE OPENING RANGE	NOMINAL CLOSED STPT. (psig)	ALLOWABLE CLOSING RANGE
B21F051D	F051D	1033	1021/1045	926	913/939
B21F051B	F051B	1073	1061/1085	936	923/949
821F047D	F0470	1113	1101/1125	946	933/959
B21F047G	F047G	1113	1101/1125	946	933/959
B21F051A	F051A	1113	1101/1125	946	933/959
B21F051F	F051F	1113	1101/1125	946	933/959