

UNITED STATES NUCLEAR REGULATORY COMMISSIONCAROLINA POWER & LIGHT COMPANYBRUNSWICK STEAM ELECTRIC PLANT, UNIT 1DOCKET NO. 50-325NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO  
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS  
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-71, issued to the Carolina Power & Light Company (CP&L, the licensee), for operation of the Brunswick Steam Electric Plant, Unit 1 (the facility), located in Brunswick County, North Carolina.

The amendment proposed would revise Section 4.6.1.2.d to allow a one-time only deferment of required Type B and C local leak rate tests (LLRTs) until the next refueling outage scheduled to begin on or before March 31, 1985 in accord with the licensee's application dated September 4, 1984, as supplemented October 22, 1984. Technical Specification (TS) Section 4.6.1.2.d requires performance of LLRTs at least once per 24-month interval based on the requirements of 10 CFR 50, Appendix J, Section D, Part 2. Therefore, an exemption to 10 CFR 50, Appendix J is also requested. A listing of the valves and penetrations involved in this request, their size (applicable to penetrations only), results of the previous LLRTs, and the current test due dates are provided in the application. These due dates range from December 1984 to March 1985. Therefore, the extensions requested range from approximately ½ month to approximately 3 ½ months. Additionally,

TS Section 4.6.1.2.f would be revised to allow a one-time only deferment of main steam line isolation valve (MSIV) leak testing until the March 31, 1985 refueling outage. The current due date for the MSIV leak testing is March 18, 1985.

The Appendix J test schedule for the facility is out-of-phase with the refueling cycle. Normally these tests are done during a refueling outage, which among other things, is desirable in order to maintain personnel exposures as-low-as-is-reasonably-achievable (ALARA). The last Unit 1 refueling outage was in late 1982 and lasted until July 1983. The fact that the Appendix J tests were done early in the refueling outage, that the outage lasted longer than originally planned and that the operating cycle was changed from 12 to 18 months caused these tests to be out-of-phase with the refueling outages. With the current LLRT schedule, mid-cycle LLRTs would need to be performed again early in Brunswick-1 Cycle 5 return to a schedule which is coincident with the Unit 1 refueling interval. Performance of mid-cycle LLRTs now and during the next fuel cycle would result in increased exposure of personnel of approximately 20 man-rem which is not consistent with CP&L's ALARA policy.

In addition, the test interval for Type C tests in Appendix J was based on two years of expected exposure of components to service conditions. In the case of the valves referred to in our request, approximately eight months of the two-year period since the valves were last tested was spent in an extended maintenance outage during which the components were not exposed to an operating environment.

Technical Specification Section 4.6.1.2.f requires that the main steam line isolation valves be leak tested at least once per 18 months. The MSIVs were last tested on May 3, 1983. Utilizing the maximum surveillance period of 125 percent, the latest required performance date is March 18, 1985. The requested extension results in only an additional 12 days, or a 1.75 percent increase, in the maximum surveillance interval permitted by the TS.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The proposed change represents a relaxation in the surveillance requirements; however, the length of the requested extension is small with respect to the maximum allowable frequency. Extending the surveillance interval for the LLRT from a maximum surveillance interval of 24 months to a range of 24½ months for some valves to as much as 27½ months for others. This does not constitute a significant reduction in the verification of

operability of the involved valves. This is based on the following information:

1. The majority of the tests requiring extensions are for valves and penetrations which are not exposed to harsh environments and typically exhibited satisfactorily test results.
2. Subsequent to the last performance of these LLRTs an eight-month outage ensued during which the valves and penetrations were not subject to normal operating conditions thus reducing the potential degradation during this period.
3. The TS limit for LLRT leakage is 159.78 SCFH (calculated in accordance with TS Section 3.6.1.2). The present recorded LLRT leakage for Brunswick-1 is approximately 58 SCFH below this limit. In addition, the TS limit of 159.78 SCFH is only 60 percent of the containment leakage calculated using 10 CFR 100 guidelines. Therefore, the present recorded LLRT for Brunswick-1 is approximately 38 percent of 10 CFR 100 guidelines.
4. Actual containment leakage during a LOCA would require leakage by two valves in series. Most LLRTs are performed between these valves, resulting in greater recorded leakage than would most probably occur during a LOCA.
5. The condition of the components is not expected to change during the requested extension period which is short in comparison with the two-year test interval.

6. The intent of Appendix J was that isolation valves be tested during refueling outages. The request is to extend the LLRT interval to coincide with the scheduled outage for refueling.

Based on the above information, operation of the facility in accord with the proposed amendment would not involve a significant increase in the probability or consequences of an accident previously evaluated.

During our review, we could find no way that the extension of time could create the possibility of a new or different kind of accident from any accident previously evaluated.

The above information, in particular items 1, 2 and 3, indicates that operation of the facility would not involve a significant reduction in a margin of safety.

Accordingly, the Commission proposes to determine that the proposed change involves no significant hazards considerations.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Comments should be addressed to the Secretary of the Commission, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attn: Docketing and Service Branch.

By November 30, 1984, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating

license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first

prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attn: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to Domenic B. Vassallo: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Executive Legal Director, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, and to George F.



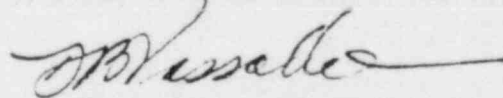
Trowbridge, Esquire, Shaw, Pittman, Potts and Trowbridge, 1800 M Street, N. W., Washington, D. C. 20036, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board designated to rule on the petition and/or request, that the petitioner has made a substantial showing of good cause for the granting of a late petition and/or request. That determination will be based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the licensee's application, dated September 4, 1984, as supplemented October 22, 1984, which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Southport, Brunswick County Library, 109 W. Moore Street, Southport, North Carolina 28461.

Dated at Bethesda, Maryland, this 24th day of October, 1984.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing