

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 1 3	PAGE (3) 1 OF 0 2
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TITLE (4)
Unexpected Steam Dump Operation Causing a Feedwater Isolation Signal

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																									
10	14	84	84	051	00	11	09	84			0 5 0 0 0																																									
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) 2</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 0 0 3</td> <td>20.402(b)</td> <td>20.406(c)</td> <td><input checked="" type="checkbox"/></td> <td>50.73(a)(2)(iv)</td> <td>73.71(b)</td> </tr> <tr> <td>20.406(a)(1)(i)</td> <td>50.36(a)(1)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(v)</td> <td>73.71(e)</td> </tr> <tr> <td>20.406(a)(1)(ii)</td> <td>50.36(a)(2)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(vii)</td> <td rowspan="3">OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td>20.406(a)(1)(iii)</td> <td>50.73(a)(2)(i)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(viii)(A)</td> </tr> <tr> <td>20.406(a)(1)(iv)</td> <td>50.73(a)(2)(ii)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(viii)(B)</td> </tr> <tr> <td>20.406(a)(1)(v)</td> <td>50.36(a)(2)(iii)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(ix)</td> <td></td> </tr> </table>												OPERATING MODE (9) 2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											POWER LEVEL (10) 0 0 3	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)	20.406(a)(1)(i)	50.36(a)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(e)	20.406(a)(1)(ii)	50.36(a)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	20.406(a)(1)(v)	50.36(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	
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LICENSEE CONTACT FOR THIS LER (12)

NAME Daniel C. Poole - Assistant Manager, Operations & Maintenance	TELEPHONE NUMBER AREA CODE 3 1 4 6 1 7 1 6 - 1 8 3 1 5 1 2
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/14/84 at 1848 CDT a Feedwater Isolation Signal (FWIS) was received when a hi-hi steam generator level signal was generated. The plant was in Mode 2 at 3% power with auxiliary feedwater supplying the steam generators at the time of the event.

Steam dump valves were in the manual mode being used for plant restoration following natural circulation testing of the Reactor Coolant System. While manipulating the steam dump valves they unexpectedly went from 20% to 40% open. This change in steam demand caused steam generator levels to swell to the hi-hi setpoint causing a FWIS. Adjusting the steam dump valves allowed the steam generator levels to return to normal operating range. The FWIS was reset at 1858.

The cause of the event has been attributed to be improper operator control of the steam dumps or sticky steam dump valve operation due to the infrequent usage of the steam dumps. Proper operation of the steam dumps was verified during testing conducted on 10/19/84 through 10/20/84.

No radioactive releases occurred and this event has in no way affected the health and safety of the public.

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PDR

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	0 5 1	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On 10/14/84 at 1848 CDT a Feedwater Isolation Signal (FWIS) was received due to a hi-hi level signal received on the steam generators. Due to startup testing, auxiliary feedwater was supplying the steam generators. The main feedwater isolation, control and bypass valves were closed, therefore, no isolations occurred as a result of the FWIS. The plant was in Mode 2 at 3% power at the time of the event.

While controlling the steam dump valves manually for plant restoration following natural circulation testing of the Reactor Coolant System, the steam dump valves unexpectedly went from 20% to 40% open. This large change in steam demand caused steam generator levels to swell to the hi-hi setpoint producing a FWIS. Upon receipt of the FWIS, the operator at the steam dump controls adjusted the steam dump valves allowing steam generator levels to return to normal operating range. The FWIS was reset at 1858.

The cause of the event has been attributed to be either operator overreaction at the steam dump controls or sticky valve operation. With the plant in early stages of operation the steam dumps have not been utilized frequently and sticky valve operation is probable during a "breaking in" period. Control Room personnel were briefed during shift turnover in reference to this event and cautioned about possible steam dump problems.

To ensure proper steam dump operation, a Dynamic Steam Dump Control Test was satisfactorily performed on 10/19/84 through 10/20/84. No subsequent steam dump problems have been encountered and no additional corrective action is deemed necessary.

No radioactive releases occurred and this event has in no way affected the health and safety of the public.

Previous occurrences: none

UNION ELECTRIC COMPANY
CALLAWAY PLANT

MAILING ADDRESS:
P. O. BOX 620
FULTON, MO. 65251

November 9, 1984

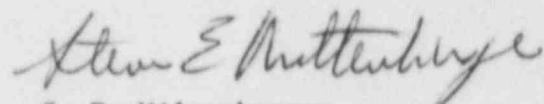
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-571

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-051-00
INADVERTENT ENGINEERED SAFETY FEATURE ACTUATION

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning an inadvertent Feedwater Isolation Signal.


S. E. Miltenberger
Manager, Callaway Plant

JTP/WRR/RCW/drs
Enclosure

cc: Distribution attached

IE22
11

cc distribution for ULNRC-971

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SEM Chrono
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