



UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 WASHINGTON, D.C. 20555

July 21, 1992

Mr. R. D. Binz IV, Chairman
 BWR Owners Group
 c/o Public Service Electric and Gas Company
 Hope Creek Generating Station
 P.O. Box 236, Mail Code N²₁
 Hancocks Bridge, NJ 08030

Dear Mr. Binz:

SUBJECT: GENERAL ELECTRIC COMPANY (GE) TOPICAL REPORT GENE-770-06-1,
 "BASES FOR CHANGES TO SURVEILLANCE TEST INTERVALS AND ALLOWED
 OUT-OF-SERVICE TIMES FOR SELECTED INSTRUMENTATION TECHNICAL
 SPECIFICATIONS"

We have completed our review of the subject topical report submitted by the BWR Owners Group by letter dated February 19, 1991. Enclosure 1 provides our Safety Evaluation Report (SER), which was prepared after reviewing the Technical Evaluation Report (TER) and responses to NRC staff questions (TER and responses to NRC staff questions attached to the SER), developed under contract by EG&G Idaho-Washington Technical Office (WTO). Our SER concurs with the findings contained in the TER.

We find that GENE-770-06-1 provides an acceptable basis for extending surveillance test intervals (STIs) and allowed outage times (AOTs) for the following actuation instrumentation:

- For the BWR4 plants: Plant Systems Actuation Instrumentation, Main Control Room Environmental Control System (MCRECS), and Safety/Relief Valves Low Low Set (LLS) Function.
- For the BWR6 plants: Plant Systems Actuation Instrumentation, Control Room Fresh Air Actuation Instrumentation (CRFA), Safety/Relief Valves, and Safety/Relief Valves Low Low Set (LLS) Function.
- For the BWR4 and BWR6 plants: End of Cycle-Recirculation Pump Trip (EOC-RPT) System Actuation Instrumentation, ATWS-RPT System Actuation Instrumentation, and Control Rod Block Instrumentation.

It should be noted that BWR4 and BWR6 RCIC System Actuation Instrumentation AOT and STI relaxations were granted in the NRC staff SER of September 13, 1991, on topical report GENE-770-06-2 (an addendum to GENE-770-06-1).

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July 21, 1992

As noted in the enclosed SER, applicants for proposed Technical Specification changes for individual plants must:

1. Confirm the applicability of the generic analyses of GENE-770-06-1 to the plant.
2. Confirm that any increase in instrument drift due to the extended STIs is properly accounted for in the setpoint calculation methodology. (For additional information on this issue, see letter from C. E. Rossi to R. F. Janecek, dated April 27, 1988.)

Enclosure 2 provides an acceptable format for proposed TS changes based on GENE-770-06-1. Our review of plant-specific changes will consider the applicability of the topical report to the specific plant.

Licenses and applicants are encouraged to propose changes to TS that are consistent with the guidance provided in the enclosures. Proposed license amendments conforming to this guidance will be expeditiously reviewed by the NRC Project Manager for the facility. Proposed amendments that deviate from this guidance and propose less stringent technical specifications will require a longer, more detailed review. Please contact the Project Manager if you have questions on this matter.

In accordance with procedures established in NUREG-0390, "Topical Reports Review Status," we request that the BWR Owners Group publish accepted revisions of GENE-770-06-1, both proprietary and non-proprietary, within three months of receipt of this letter. The accepted versions should (1) incorporate this letter and the enclosed Safety Evaluation Report including the Technical Evaluation Report, between the Title page and the abstract and (2) include an -A (designated accepted) following the report identification symbol.

Should our acceptance criteria or regulations change so that our conclusions as to the acceptability of the report are no longer valid, the BWR Owners Group and/or the applicants referencing this topical report will be expected to revise and resubmit their respective documentation, or submit justification for the continued applicability of the topical report without revision of its documentation.

Sincerely,



Charles E. Rossi, Director
Division of Operational Events Assessment
Office of Nuclear Reactor Regulation

Enclosures

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 Original Signed By
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 Division of Operational Events Assessment
 Office of Nuclear Reactor Regulation

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 C.E.Rossi
 7/2/92

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*(see memo to WTRussell dated July 20, 1992
for enclosure)