NRC FORM 366 (7-77)

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U.S. NUCLEAR REGULATORY COMMISSION

	(7~77)	LICENSEE EVENT REPORT EXHIBIT A
		CONTROL BLOCK: 1_1_1_1_1_1_1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
	$\frac{1}{7} \frac{1}{1} \frac{1}{8}$	1 9 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1
1	$\frac{1}{7} \frac{0}{8} \frac{1}{8}$	REPORT     I
	10121	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10 [During Mode 1 operation, following routine draining operation of the Reactor Drain Tank, outside containment [
	10131	lisolation valve 2CV-2201-2 would not fully close. The affected penetration was isolated within one hour by
	10141	Ideactivating the inside containment isolation valve 2CV-2202-1 in the closed position, meeting the requirements
1	10151	of Action Statement Technical Specification (T.S.) 3.6.3.1.b. Other occurrences on containment penetrations
1	10161	lare 50-368/79-063, 79-061, 79-044, & 78-19. This occurrence is reportable per T.S. 6.9.1.9.b.
	10171	I
	10181	11
	1 <u>0</u> 1 <u>9</u> 1 7 8	SYSTEM CAUSE CAUSE CAUSE COMP VALVE 80   CODE CODE SUBCODE COMPONENT CODE SUBCODE
1	17	$\begin{array}{c ccccccccccccccccccccccccccccccccccc$
1	A( T)	$\begin{array}{c ccccccccccccccccccccccccccccccccccc$
÷	1101	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27
		Investigation revealed foreign material had damaged seats and ball causing value to leak through. The value investigation revealed, cleaned and reassembled. No replacement parts were available, the inside containment investigation of the second se
	11121	lisolation valve, 2CV-2202-1 remained deactivated in the closed position. A subsequent engineering evaluation
	11131	Iresulted in replacement of the EPG ball valve and Matryx actuator with a Canadian Worchester Controls pneumatic
	114	[actuated valve package during the 1983 refueling outage. 2CV-2201-2 was released to operations (Continued)
		FACILITY     METHOD OF     80       STATUS     % POWER     OTHER STATUS     DISCOVERY     DISCOVERY DESCRIPTION       1     B     128     1     0     1     1     1     NA     132
		ACTIVITY     CONTENT     80       RELEASED     OF RELEASE     AMOUNT OF ACTIVITY     LOCATION OF RELEASE       1     2     133     1     2       9     10     11     44     45
		9 10 11 44 45 130   PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 80   1 0 137 1 138 1   9 11 12 13 139   ERSONNEL INJURIES 80
	$\frac{1}{7} \frac{1}{8} \frac{1}{8}$	NUMBER     DESCRIPTION       1     0     10     140     141       9     11     12     80
	11191	LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 1 Z 142 1 NA IEZZ 143
	1 <u>2101</u> 7 8	PUBLICITY     NRC USE ONLY       ISSUED     DESCRIPTION       I     NRC USE ONLY       I     NRC USE ONLY       9     10       68     69
		NAME OF PREPARER:Patricia CampbellPHONE:PHONE:
	850311 PDR A S	0059 850225 DOCK 05000368 PDR .

NRC FORM 366 (7-77)

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LICENSEE EVENT REPORT

ATTACHMENT

U.S. NUCLEAR REGULATORY COMMISSION

EXHIBIT A

LER NO. 50-368/79-066/03X-1

Cause Description and Corrective Actions (Continued)

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on 12/06/83, and operation has been satisfactory since that time.



## ARKANSAS POWER & LIGHT COMPANY POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

February 25, 1985

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U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

> Subject: Arkansas Nuclear One - Unit 2 Docket No. 50-368 License No. NPF-6 Licensee Event Report No. 79-066/03X-1

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 2 Technical Specification 6.9.1.9.b, attached is the subject report concerning the failure of outside containment isolation valve 2CV-2201-2 to close properly. This is a revision to a previous submittal dated August 31, 1979.

Very truly yours,

J. Ted Enos Manager, Licensing

JTE: RJS: ds

Attachment

cc: Mr. Richard C. DeYoung Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission Washington, DC 20555

> Mr. Norman M. Haller, Director Office of Management & Program Analysis U. S. Muclear Regulatory Commission Washington, DC 20555

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