

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
LaSalle County Station Unit 1

DOCKET NUMBER (2)
0 5 0 0 0 3 1 7 3

PAGE (3)
1 OF 0 3

TITLE (4)
Scram Due to MSIV Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
2	0	28	5	8	5	0	2	28	5	NA	0 5 0 0 0
2	0	28	5	8	5	0	2	28	5	NA	0 5 0 0 0

OPERATING MODE (9) 1

POWER LEVEL (10) 9916

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11)

20.402(b)	20.405(a)	20.726(a)(2)(iv)	73.71(b)
20.405(a)(1)(B)	20.39(a)(1)	20.726(a)(2)(v)	73.71(c)
20.405(a)(1)(B)	20.39(a)(2)	20.726(a)(2)(vB)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
20.405(a)(1)(B)	20.726(a)(2)(i)	20.726(a)(2)(vB)(A)	
20.405(a)(1)(D)	20.726(a)(2)(B)	20.726(a)(2)(vB)(B)	
20.405(a)(1)(F)	20.726(a)(2)(B)	20.726(a)(2)(v)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: John Ullrich, extension 571

TELEPHONE NUMBER: 8115 3517-161611

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	VA	PISV	A499	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-space typewritten lines) (16)

On February 2, 1985 at 1200, a Group I isolation occurred on Unit 1 due to high main steam tunnel ambient air temperature. This isolation resulted in a reactor scram.

At the time of this event, Unit 1 was at 96% power. Reactor Building Ventilation (VA) had been shut down for the repair of the solenoid valves on isolation dampers, 1VR05YA and 1VR05YB. Due to the lack of ventilation flow, the ambient air temperature in the main steam tunnel increased to the trip setpoint of the Leak Detection system (IJ) resulting in a Group 1 isolation (JM).

Following the scram, Reactor pressure was maintained by the "U" SRV (SB) on the Low-Low Setpoint. Logic and Reactor level was maintained by Reactor Core Isolation Cooling (BN) and initially by the Motor Driven Reactor Feedwater Pump (SJ). All plant systems responded as designed for this type event. Safe operation of the plant was maintained at all times.

Upon completion of isolation damper 1VR05YA and 1VR05YB repair, the Reactor Building Ventilation system was restarted to cool down the mainsteam tunnel. When the Group 1 isolation signal cleared, pressure was equalized around and the MSIV's were opened. A normal plant recovery was then commenced.

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		8 5	- 0 1 1	- 0 0	0	2	OF 0 3

TEXT IF more space is required, use additional NRC Form 2864's (17)

I. EVENT DESCRIPTION

At 1200 on February 2, 1985, a Group one (1) Isolation (full main steam isolation valve and main steam line drain valve closure) occurred on Unit 1 due to high main steam tunnel ambient temperature. This isolation resulted in a reactor scram. In addition to the initial scram, six additional scram signals were received as a result of the vessel water level fluctuations during safety/relief valve actuations.

II. Cause

At the time of this event, Unit 1 was operating at 96% power. The Unit 1 Reactor Building Ventilation system (VA) had been shut down for the repair of the solenoid valves for isolation dampers 1VR05YA and 1VR05YB. (Refer to LER 373/85-08-00.) Due to the lack of ventilation flow, the ambient air temperature in the main steam tunnel increased to the trip setpoint of the Leak Detection system (IJ), resulting in a Group 1 isolation (JM). Because of the isolation signal, the main steam isolation valves (MSIV, SB) closed which resulted in a reactor scram (JC).

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

Upon receipt of the high main steam tunnel ambient air temperature signal, a Group 1 isolation signal occurred, closing all of the main steam isolation valves and the main steam line drain valves, which resulted in a reactor scram. Reactor pressure was controlled by the "U" safety/relief valve (SRV) cycling (16 times) by the Low-Low Setpoint Logic and reactor level was initially maintained manually at approximately 22 inches, by the Reactor Core Isolation Cooling system (RCIC, BN) and the Motor Driven Reactor Feed Pump (SJ). Due to feedwater regulating valve leakage, the Motor Driven Reactor Feedwater Pump was eventually turned off due to reactor water level increase, and level was then controlled solely by RCIC and CRD (AA) flow. During the SRV actuation, reactor water level would swell 10-20 inches, then upon SRV closure the water level would drop 15-20 inches below the initial water level, sometimes reaching the Level 3 (+12.5 inches) setpoint causing another scram. This resulted in receiving six (6) additional scram signals. The reason for this large level drop, below the initial water level, was due to the water lost from the reactor to the Suppression Pool (NH) through the SRV. All plant equipment responded as designed for this type event. No ECCS systems were required for this event. Upon review of the event, reactor vessel level and pressure responded as would be expected for a transient of this type. The multiple scrams which occurred are not believed to have caused any difficulties with regards to the control rod drives.

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		85	011	010	03	OF	03

TEXT (If more space is required, use additional NRC Form 388A's) (17)

IV. CORRECTIVE ACTION

Upon completion of repairs to the Reactor Building ventilation isolation dampers, 1VR05YA and 1VR05YB, the Reactor Building Ventilation system was restarted to cool down the main steam tunnel. When the Group 1 isolation signal was cleared, pressure was equalized around the MSIV's and the MSIV's were opened. A normal plant recovery was then commenced. In addition, Action Item Records were written for the following: 1) AIR 01-85-67020 tracks Operating Procedure changes to allow for temporarily jumpering out the main steam tunnel high ambient air and high differential temperature isolation signals when securing Reactor Building ventilation; 2) AIR 01-85-67019 tracks pursuing a Technical Specification change to allow the ambient air temperature detectors to be inoperable the same length of time as the differential temperature detectors; 3) AIR 01-85-67021 tracks initiation of a study to determine the feasibility of raising the actuation setpoint of the main steam tunnel ambient air temperature detectors; 4) AIR 01-85-67023 tracks a study to evaluate feedwater regulating valve performance; 5) CRD stall flows will be monitored to determine if any problems with regards to the CRD's has resulted from the multiple scram signals. AIR 1-85-67032

V. PREVIOUS OCCURRENCES

None.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

John Ullrich, 815/357-6761, extension 571.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

February 22, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #85-011-00, Docket #050-373 is being submitted to your office in accordance with 10CFR50.73.

for *R. D. Bishop*
G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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