

ATTACHMENT 1

PROPOSED OPERATING LICENSE AMENDMENT

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PDR ADOCK 05000338  
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TABLE 4.11-2 (Continued)

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) <sup>a</sup> (uCi/ml)
E. Condenser Air Ejector Vent <sup>f</sup>	W Grab Sample	W	Principle Gamma Emitters <sup>g</sup>	$1 \times 10^{-4}$
Steam Generator Blowdown Vent <sup>f</sup>			H-3	$1 \times 10^{-6}$
F. Containment Vacuum Steam Ejector (Rogger)	P Grab Sample	P	Principle Gamma Emitters <sup>b</sup>	$1 \times 10^{-4}$
			H-3	$1 \times 10^{-6}$

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

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3.11.2.5 The concentration of oxygen in the waste gas decay tanks shall be limited to less than or equal to 2% by volume whenever the hydrogen concentration exceeds 4% by volume and is less than 96% by volume.

APPLICABILITY: At all times.

ACTION:

- a. With the concentration of oxygen in the waste gas decay tanks greater than 2% by volume but less than or equal to 4% by volume, reduce the oxygen concentration to the above limits within 48 hours.
- b. With the concentration of oxygen in the waste gas decay tanks greater than 4% volume and the hydrogen concentration greater than 2% by volume, immediately suspend all additions of waste gases to the system and reduce the concentration of oxygen to less than or equal to 2% by volume without delay.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

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4.11.2.5 The concentrations of hydrogen and oxygen in the waste gas decay tanks shall be determined to be within the above limits by continuously monitoring the waste gases in the waste gas decay tanks with the hydrogen and oxygen monitors required OPERABLE by Table 3.3-14 of Specification 3.3.3.11.

## 5.0 DESIGN FEATURES

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### 5.1 SITE

#### EXCLUSION AREA

5.1.1 The exclusion area shall be as shown in Figure 5.1-1.

#### LOW POPULATION ZONE

5.1.2 The low population zone shall be as shown in Figure 5.1-2.

#### MAP DEFINING UNRESTRICTED AREAS FOR RADIOACTIVE GASEOUS AND LIQUID EFFLUENTS

5.1.3 Information regarding radioactive gaseous and liquid effluents, which allows identification of structures and release points as well as definition of UNRESTRICTED AREAS within the SITE BOUNDARY that are accessible to MEMBERS OF THE PUBLIC, shall be as shown in Figure 5.1-1.

### 5.2 CONTAINMENT

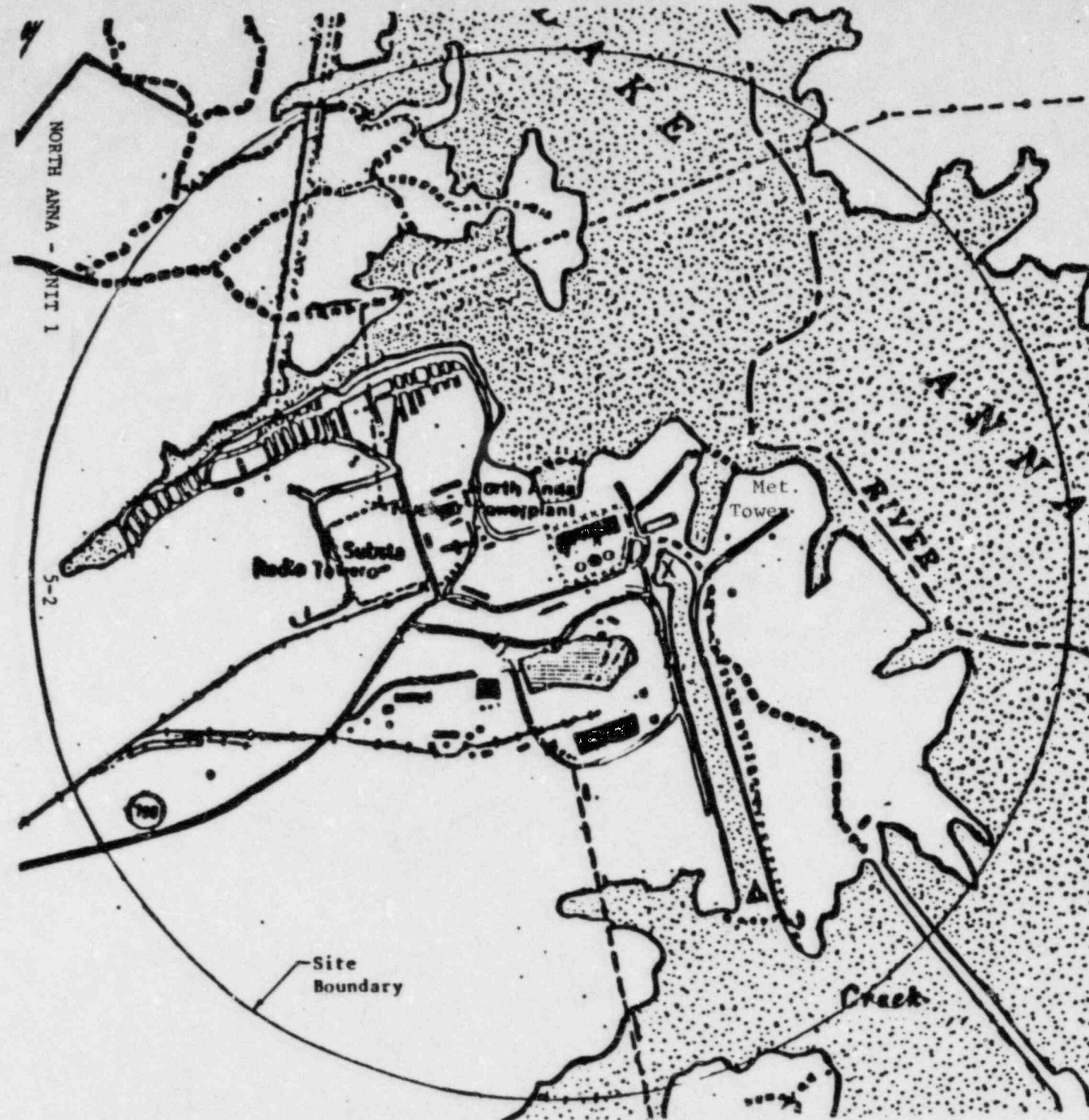
#### CONFIGURATION

5.2.1 The reactor containment building is a steel lined, reinforced concrete building of cylindrical shape with a dome roof and having the following design features:

- a. Nominal inside diameter = 126 feet.
- b. Nominal inside height = 190 feet, 7 inches.
- c. Minimum thickness of concrete walls = 4.5 feet.
- d. Minimum thickness of concrete roof = 2.5 feet.
- e. Minimum thickness of concrete floor pad = 10 feet.
- f. Nominal thickness of the cylindrical portion of the steel liner = 3/8 inches.
- g. Net free volume =  $1.825 \times 10^6$  cubic feet.
- h. Nominal thickness of hemispherical dome portion of the steel liner = 1/2 inch.

#### DESIGN PRESSURE AND TEMPERATURE

5.2.2 The reactor containment building is designed and shall be maintained for a maximum internal pressure of 45 psig and a temperature of 280°F.



- Gaseous Release
  1. Process Vent- 157.5 ft.
  2. Vent-Vent A&B and other release points considered ground level releases

X Liquid Release to the Discharge Canal

▲ Liquid Release to the Unrestricted Area

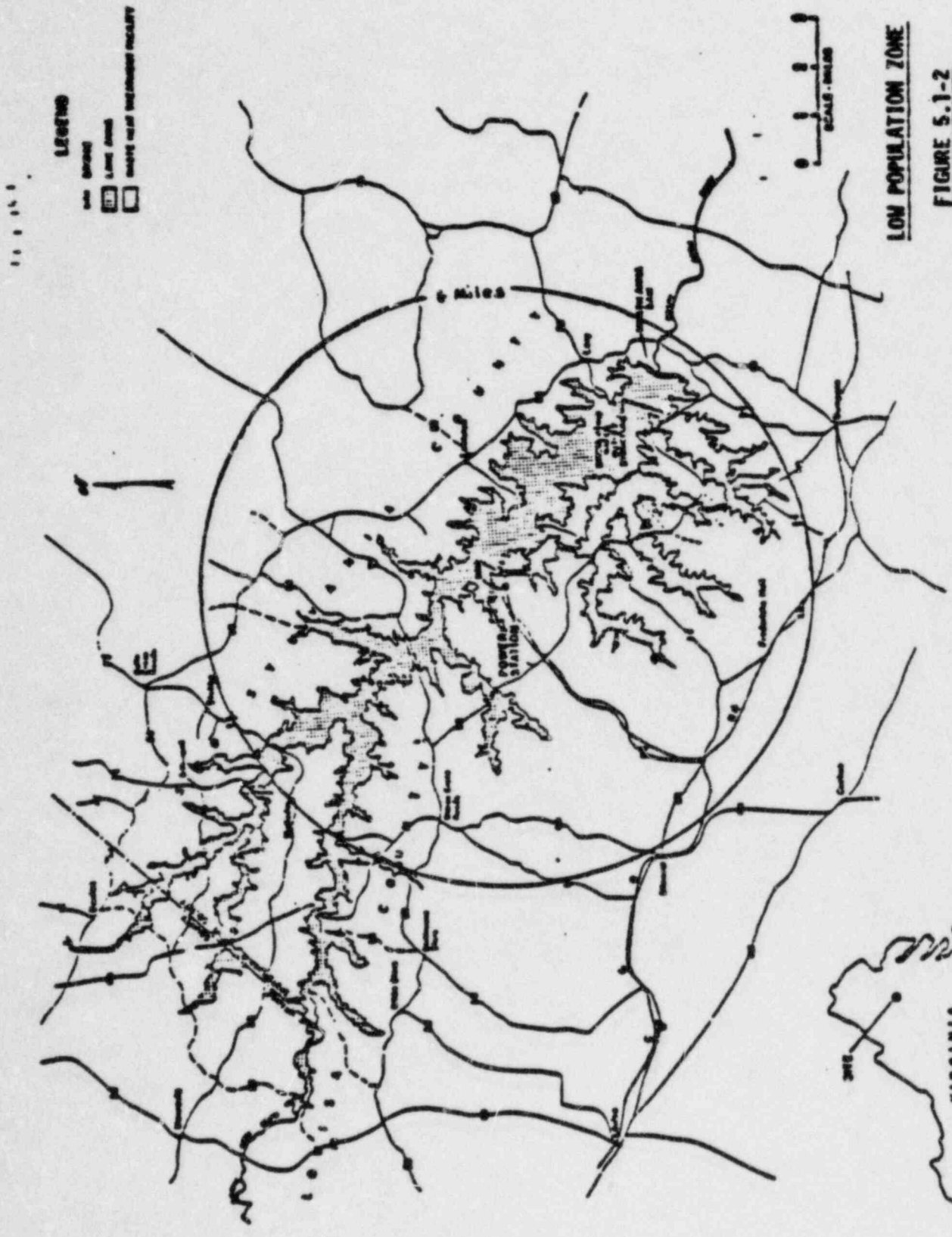
●●● Buoy Barriers

--- Security Fence- Area outside is unrestricted for gaseous effluents

Land Maximum Member of the Public Occupancy = 336 hrs/year

Lake Maximum Member of the Public Occupancy = 2232 hrs/year

**Figure 5.1-1**  
**Map Defining Unrestricted Areas for Radioactive Gaseous and Liquid Effluents**



**LOW POPULATION ZONE**

**FIGURE 5.1-2**

ATTACHMENT 2

TABLE 4.11-2 (Continued)

## RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) <sup>a</sup> (uCi/ml)
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F. Containment Vacuum Steam Ejector (Hogger)	P Grab Sample	P	Principle Gamma Emitters <sup>b</sup>	$1 \times 10^{-4}$
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## 5.0 DESIGN FEATURES

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#### DESIGN PRESSURE AND TEMPERATURE

5.2.2 The reactor containment building is designed and shall be maintained for a maximum internal pressure of 45 psig and a temperature of 280°F.



**Figure 5.1-1**  
**Map Defining Unrestricted Areas for Radioactive Gaseous and Liquid Effluents**

11 1 1 1 1

**LEGEND**

● SITE

▨ 5.000 ACRES

▩ 1.000 ACRES

○ 500 ACRES

○ 250 ACRES

○ 100 ACRES

○ 50 ACRES

○ 25 ACRES

○ 10 ACRES

○ 5 ACRES

○ 2 ACRES

○ 1 ACRES

○ 0.5 ACRES

○ 0.25 ACRES

○ 0.1 ACRES

○ 0.05 ACRES

○ 0.025 ACRES

○ 0.01 ACRES

○ 0.005 ACRES

○ 0.0025 ACRES

○ 0.001 ACRES

○ 0.0005 ACRES

○ 0.00025 ACRES

○ 0.0001 ACRES

○ 0.00005 ACRES

○ 0.000025 ACRES

○ 0.00001 ACRES

○ 0.000005 ACRES

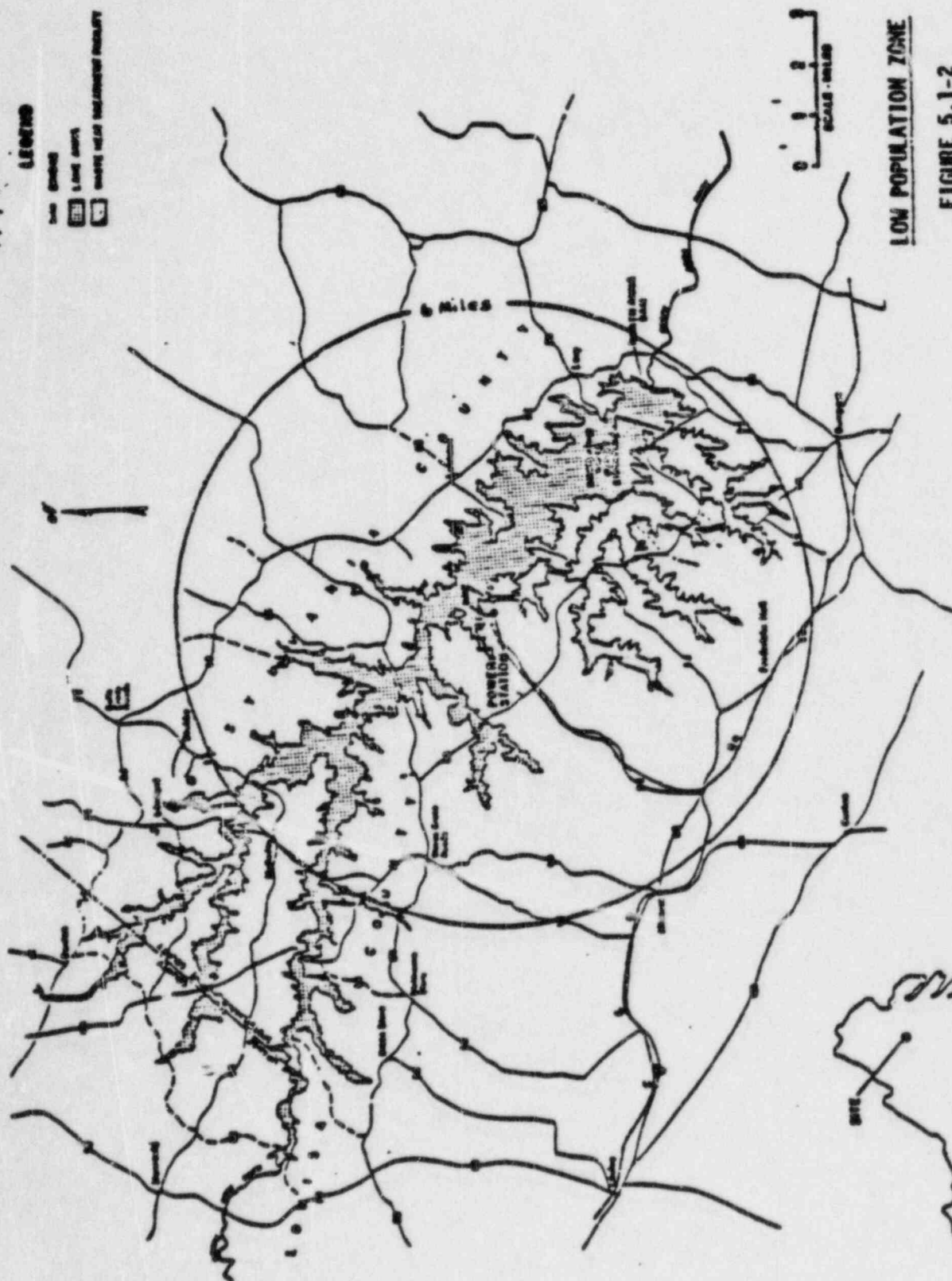
○ 0.0000025 ACRES

○ 0.000001 ACRES

○ 0.0000005 ACRES

○ 0.00000025 ACRES

○ 0.0000001 ACRES



**LOW POPULATION ZONE**

**FIGURE 5.1-2**

## ADMINISTRATIVE CONTROLS

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- g. Records of reactor tests and experiments.
- h. Records of training and qualification for current members of the plant staff.
- i. Records of in-service inspections performed pursuant to these Technical Specifications.
- j. Records of Quality Assurance activities required by the QA Manual.
- k. Records of the service life of all hydraulic and mechanical snubbers listed in Tables 3.7-4a and 3.7-4b including the date at which the service life commences and associated installation and maintenance records.
- l. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- m. Records of meetings of the SNSOC.
- n. Records of meetings of the System Nuclear Safety and Operating Committee to issuance of Amendment No. 11.
- o. Records of secondary water sampling and water quality.
- p. Records for Environmental Qualification which are covered under the provisions of Paragraph 2 C(4)(e) of License No. NPF-7.
- q. Records of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This would include procedures effective at specified times and QA records showing that these procedures were followed.

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance

ATTACHMENT 3

## DISCUSSION OF PROPOSED TECHNICAL SPECIFICATION CHANGES

The proposed change to North Anna 1 and 2 Technical Specifications Table 4.11 - 2 will include the Containment Vacuum Steam Ejector (Hogger) as a gaseous release path that should be monitored. A containment air sample will be sampled by grab sample and have a minimum analysis frequency of prior to each release, for each ejector. This release path was inadvertently left out of our RETS submittals of December 16, 1982 and February 25, 1983 and was since approved by the NRC on May 5, 1983, in Amendment Nos. 48 and 31, for North Anna 1 and 2, respectively. Since this proposed change was left out of our previous submittals, and it should have been included, and it has since been approved by the NRC, the change should be administratively inserted.

The proposed change to North Anna 1 and 2 Technical Specifications 5.1.2 is to indicate that the low population zone is shown in Figure 5.1-2 instead of Figure 5.1-1. Amendment Nos. 48 and 31, for North Anna 1 and 2, respectively, deleted Figure 5.1-2. Figure 5.1-2 should be re-inserted into the Technical Specifications and so indicated in Technical Specification 5.1.2.

The proposed change to North Anna 1 and 2 Technical Specifications Figure 5.1-1 should be revised to include the location of the Met. Tower. The location of the Met. Tower was inadvertently left out of our RETS submittals.

The proposed change to North Anna 1 Technical Specification 4.11.2.5 is to revise the table number that is referenced in the surveillance requirement. Table 3.3-13 is referenced and the correct table number should be Table 3.3-14.

The proposed change to North Anna 2 Technical Specification 6.10.2n is to make the requirement read, "Records of meetings of the System Nuclear Safety and Operating Committee to issuance of Amendment No. 11." The Amendment No. (11) was inadvertently left off our RETS submittal and has since been approved by the NRC.

These proposed changes do not pose a significant hazards consideration as described in the Federal Register, dated April 6, 1983, Page 14870, Example (i); a purely administrative change to the Technical Specifications: for example, a change to achieve consistency throughout the Technical Specifications. The proposed changes are needed to correct various administrative problems that have developed since our RETS submittals of December 17, 1982 and February 25, 1983 and the issuance of Amendment Nos. 48 and 31, for North Anna 1 and 2, respectively, which dealt with RETS. A number of corrections needed to be made due to inadvertent discrepancies from our RETS submittals.