



Commonwealth Edison

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September 24, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Byron Generating Station Units 1 and 2
Braidwood Generating Station Units 1 and 2
NUREG-0737, Supplement 1 Requirements for
Emergency Response Capability (Generic
Letter No. 82-33)
NRC Docket Nos. 50-454/455 and 50-456/457

- References (a): December 17, 1982 letter from D. G. Eisenhut to Licensees and Applicants.
- (b): December 6, 1983 letter from Cordell Reed to H. R. Denton.

Dear Mr. Denton:

This letter provides updated implementation schedules for specific improvements in emergency response capability required to satisfy the requirements of NUREG-0737, Supplement 1, which were imposed in reference (a). These updated schedules supersede all previous schedules, the last of which were provided in reference (b).

Attachment A to this letter contains the updated schedules for completion of the NUREG-0737, Supplement 1 requirements. Substantive changes have been highlighted with a vertical bar in the righthand margin. A calendar date has been specified for operability of the Byron 1 Safety Parameter Display System (SPDS) rather than the deadline of commercial service used previously. The Byron 2 and Braidwood SPDS's will be operational by fuel load. The implementation schedules for the revised EOP's have also been respecified based on an understanding that the NRC will be approving revision 1 of the EPG's in late 1984. The implementation of the A-model has been postponed to provide adequate time to implement the necessary hardware and software changes. Adequate alternate dose calculation techniques are available as described in reference (b). Please address questions regarding these changes to this office.

One signed original and fifteen copies of this letter and the attachment are provided for NRC review.

Very truly yours,

T. R. Tramm

Nuclear Licensing Administrator

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cc: Byron Resident Inspector

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ATTACHMENT A

Revised Section 9 Byron/Braidwood Schedules

SECTION 9 SCHEDULES

EMERGENCY RESPONSE CAPABILITY

BYRON

BRAIDWOOD

I. SAFETY PARAMETER DISPLAY SYSTEM (SPDS)

a. Submit Safety Analysis

- | | | |
|---|----------|----------|
| 1) Criteria for parameter selection including V&V description | 12-29-83 | 12-29-83 |
| 2) HFR of data display and functions (assumes NRC acceptance of DCRDR final summary report within 2 months) | 08-01-87 | 08-01-87 |
| 3) Verify parameter selection (assumes NRC acceptance of DCRDR final summary report within 2 months) | 08-01-87 | 08-01-87 |

b. SPDS Operational - Defined as design, hardware and software installation, functional testing, and initial operator training complete

03-30-85	Prior to fuel load
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c. Modifications to SPDS resulting from DCRDR, SPDS HFR, or RG 1.97

- | | | |
|--------------------|----------|----------|
| 1) Submit schedule | 08-01-87 | 08-01-87 |
|--------------------|----------|----------|

II. DETAILED CONTROL ROOM DESIGN REVIEW (DCRDR)

- | | | |
|--------------------------------|----------|----------|
| a. Submit Program Plan | 04-14-83 | 04-14-83 |
| b. Submit Final Summary Report | 12-01-86 | 12-01-86 |

III. REGULATORY GUIDE 1.97 - REVISION 2

- | | | |
|---|----------|----------|
| a. Submit a preliminary report describing how the requirements of Reg. Guide 1.97 specified in Supplement 1 to NUREG-0737 have been or will be met. | 03-01-87 | 03-01-87 |
|---|----------|----------|

EMERGENCY RESPONSE CAPABILITY

BYRON

BRAIDWOOD

- b. Submit a final report including a schedule for installation (assumes NRC acceptance of DCRDR results within 3 months of submittal date of DCRDR summary report for that Station).

09-01-87

09-01-87

IV. UPGRADE EMERGENCY OPERATING PROCEDURES (EOPs)

- a. Submit Procedures Generation Package for WOG EPG Revision 1.

3 months from NRC approval of Revision 1 guidelines.

3 months from NRC approval of Revision 1 guidelines.

- b. Implement upgraded EOPs based upon Revision 1 guidelines.

12 months from NRC approval of Revision 1 guidelines

fuel load

V. EMERGENCY RESPONSE FACILITIES

- a. Technical Support Center (TSC)

- 1) Complete with the exception of impact due to RG 1.97 and HFR
- 2) Modifications to TSC resulting from RG 1.97 and HFR (assumes NR acceptance of DCRDR results with months of submittal date of DCRD summary report for that station)

Prior to Fuel Load

Prior to Fuel Load

- a. Submit schedule

06-01-87

06-01-87

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SECTION 9 SCHEDULES

EMERGENCY RESPONSE CAPABILITY

BYRON

BRAIDWOOD

b.	Operational Support Center (OSC)	Prior to Fuel Load	Prior to Fuel Load
c.	Emergency Operations Facility (EOF)		
1)	Complete with the exception of impact due to RG 1.97 and HFR, and A-model, and industrial security	Prior to Fuel Load	Prior to Fuel Load
a.	Modifications to EOF resulting from RG 1.97 and HFR		
1)	Submit schedule	06-01-87	06-01-87
b.	Implement A-model	01-01-86	Prior to Fuel load
c.	Implement Industrial Security including remote alarm	Prior to Fuel Load	Prior to Fuel Load

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