



Commonwealth Edison
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DMB

September 18, 1984

Mr. James G. Keppler
Regional Administrator
U. S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Subject: Braidwood Station Units 1 and 2
10 CFR 50.55(e) Interim Report
Piping Wall Thickness Deficiencies
NRC Docket Nos. 50-456/457

Reference (a): E. D. Swartz letter to J. G. Keppler
dated July 20, 1984

Dear Mr. Keppler:

On June 21, 1984, the Commonwealth Edison Company notified your office of a deficiency reportable pursuant to 10 CFR 50.55(e) regarding wall thickness inadequacies for small bore ASME Class II piping at our Braidwood Station. Reference (a) provided information concerning this matter to fulfill the thirty day reporting requirement. This deficiency was assigned Number 84-10 for tracking purposes. This letter is to provide an updated status of the corrective actions taken to further investigate this issue and is considered to be an Interim Report.

INVESTIGATION RESULTS

In Reference (a), it was reported that investigations to date indicated that the pipe wall thickness deficiencies appeared to be limited to two inch schedule 80 carbon steel pipe, heat number KD6751. This lot of pipe was stored outdoors for an extended period of time, became corroded, and was subsequently chemically cleaned. It should be noted that the chemical cleaning was conducted by a non safety-related vendor.

Additional investigations and measurements of other two inch and under carbon steel pipe which was subjected to similar conditions have been conducted. Results indicate that a minimum of two other heat number combinations may also have wall thicknesses less than those required by material specifications.

ADDITIONAL CORRECTIVE ACTIONS TAKEN

As a result of these findings, all uninstalled two inch and under carbon steel pipe that was subjected to prolonged outdoor exposure and chemically cleaned has been placed on hold. Further

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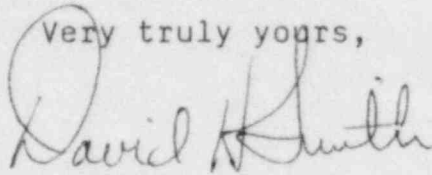
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investigation of this issue is being pursued by the Commonwealth Edison Company. A supplemental report will be sent to the NRC by November 1, 1984, updating our efforts to determine the scope and impact of this issue. This report will indicate the progress of the Sargent and Lundy investigation described in Reference (a).

Please address any questions that you or your staff may have concerning this matter to this office.

Very truly yours,

A handwritten signature in cursive script, appearing to read "David H. Smith".

David H. Smith
Nuclear Licensing Administrator

cc: NRC Resident Inspector - Braidwood

Director of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555