

GPU NUCLEAR CORPORATION
OYSTER CREEK NUCLEAR GENERATING STATION

PROVISIONAL OPERATING
LICENSE NO. DPR-16

TECHNICAL SPECIFICATION
CHANGE REQUEST NO. 130
DOCKET NO. 50-219

Applicant submits by this Technical Specification Change Request No. 130 to the Oyster Creek Nuclear Generating Station Technical Specifications, changes to Section 4.5.F.1.b.

By

Peter B. Fiedler
Peter B. Fiedler
Vice President and Director
Oyster Creek

Sworn and subscribed to before me this 25 day of September 1984.

Janice L. Bondmore
A Notary Public of NJ

JANICE L. BONDEMORE
NOTARY PUBLIC OF NEW JERSEY
My Commission Expires July 31, 1985

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

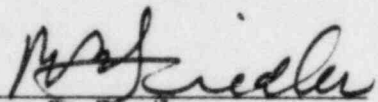
IN THE MATTER OF)
)
GPU NUCLEAR CORPORATION)

DOCKET NO. 50-219

CERTIFICATE OF SERVICE

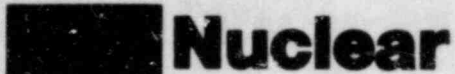
This is to certify that a copy of Technical Specification Change Request No. 130 for the Oyster Creek Nuclear Generating Station Technical Specifications, filed with the United States Nuclear Regulatory Commission on September 25, 1984, has this day of September 25, 1984, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Henry A. Delo, Jr.
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

By 
Peter B. Fiedler
Vice President and Director
Oyster Creek

DATED: September 25, 1984

PBF:dam



GPU Nuclear Corporation
Post Office Box 388
Route 9 South
Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

September 25, 1984

The Honorable Henry A. Delo, Jr.
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

Dear Mayor Delo:

Enclosed herewith is one copy of the Technical Specification Change Request No. 130 for the Oyster Creek Nuclear Generating Station Operating License.

This document was filed with the United States Nuclear Regulatory Commission on September 25, 1984.

Very truly yours,

A handwritten signature in dark ink, appearing to read "P. B. Fiedler".

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dam
Enclosure

OYSTER CREEK NUCLEAR GENERATING STATION
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 130

Applicant hereby requests the Commission to change Appendix A to the above captioned license as follows:

1. Section to be changed:

4.5.F.1.b

2. Extent of change:

Remove the percentage requirements for specific Type B and C tests and replace with an overall combined Type B and C acceptance criteria.

3. Discussion:

On August 7, 1975, the NRC requested Jersey Central Power and Light (JCP&L) to review the containment leakage testing program for the Oyster Creek Nuclear Generating Station and to provide a plan for achieving full compliance with the requirements of 10 CFR 50, Appendix J.

By JCP&L letter dated December 24, 1975, as supplemented by letters dated August 12, 1976, November 22, 1978 and June 27, 1980, JCP&L submitted their proposed plans for meeting the Appendix J criteria. On March 4, 1982, the NRC issued the Safety Evaluation for the Appendix J review, finding the analysis submitted by JCP&L to be acceptable.

The next step was for JCP&L (now the responsibility of GPU Nuclear) to develop and submit the proposed Technical Specifications. While this submittal is in the process of being written, there is a change that must be made now in order for Oyster Creek to successfully complete their Appendix J testing during this cycle 10 outage.

The Oyster Creek Technical Specifications, Section 4.5.F.1.b specifies allowable limits for double gasketed seals, testable penetrations and isolation valves, and primary containment air purge penetrations and reactor building to torus vacuum relief valves. Together, these limits comprise the acceptance criteria for the combined leakage rate of all penetrations and isolation valves subject to Type B and C tests which must be less than 0.60 of the maximum allowable limit (L_a) adjusted to 20 psig. As a result of our commitment to Type C test additional isolation valves, it has become difficult to meet the 30% $L_{t0}(20)$ limitation for testable penetrations and isolation valves.

In reviewing Specification 4.5.F.1.b, it has been determined that acceptance should not be based on the individual leakage of the various types of penetrations and valves, but should be based on the combined leakage of all penetrations and valves which are required to be Type B and C tested.

GPU Nuclear is making an effort to revise the Oyster Creek Technical Specifications to incorporate the Appendix J requirements. This effort is nearing completion but, it is important that we now submit this advance change to bring the acceptance criteria for the combined leakage rate of all penetrations and valve subject to Type B and C tests up-to-date.

This proposed change request is consistent with the intent for meeting the 10 CFR 50, Appendix J requirements as outlined in our submittals, the NRC safety evaluation of these submittals, and the acceptance criteria as stated in 10 CFR 50, Appendix J, Section III.C.3.

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Pursuant to 10 CFR 50.91, an analysis concerning significant hazards considerations is provided below:

1. Section to be changed:

4.5.F.1.b

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This proposed change request is consistent with the intent for meeting the 10 CFR 50, Appendix J requirements as outlined in our submittals, the NRC safety evaluation of these submittals, and the acceptance criteria as stated in 10 CFR 50, Appendix J, Section III.C.3.

We have determined that this change request involves no significant hazards considerations in that operation of the Oyster Creek Plant in accordance with the proposed amendment will not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated; or
2. Create the probability of a new or different kind of accident from any accident previously evaluated; or
3. Involve a significant reduction in a margin of safety.