



**Wisconsin Electric** POWER COMPANY  
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

September 25, 1984

Mr. H. R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. NUCLEAR REGULATORY COMMISSION  
Washington, D. C. 20555

Attention: Mr. J. R. Miller, Chief  
Operating Reactors, Branch 3

Gentlemen:

DOCKETS 50-266 AND 50-301  
MODIFICATION TO TECHNICAL SPECIFICATION CHANGE REQUEST NO. 88  
HEAVY LOAD RESTRICTIONS, SAFETY-RELATED SNUBBERS,  
AND MISCELLANEOUS ADMINISTRATIVE CHANGES  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

In accordance with Sections 50.59 and 50.90 of 10 CFR 50, Wisconsin Electric Power Company (Licensee) modifies its application for an amendment to Facility Operating Licenses DPR-24 and DPR-27 as filed by letter dated March 16, 1984.

This modification deletes the listing of safety-related shock suppressors (snubbers), Table 15.3.13-1, and references thereto in specifications in accordance with NRC Generic Letter 84-13, "Technical Specification for Snubbers", dated May 3, 1984. Additionally, Specification 15.4.13.2 has been changed to require the functional testing of a representative sample of approximately 10 percent of the safety-related snubbers. This change more accurately reflects the intent of the requirement of functional testing and makes the specification independent of the number of safety-related snubbers.

This modification also incorporates recent organizational changes for senior management of Wisconsin Electric Power Company. Specifications 15.6.5.1, "Manager's Supervisory Staff", 15.6.5.3, "Off-Site Review Committee", 15.6.6, "Reportable Occurrence Action", and 15.6.7, "Action to be Taken if a Safety Limit is Exceeded", and Figures 15.6.2-1 and 15.6.2-3 have been changed to reflect the creation of the position of Vice Chairman of the Board and elimination of the position of Executive Vice President.

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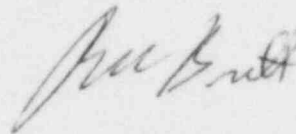
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Lastly, this modification clarifies our previous Technical Specification change request as discussed with the NRC Point Beach Project Manager, Mr. T. Colburn.

As required by 10 CFR 50.91(a)(1), we have prepared the following discussion concerning the issue of whether or not the proposed modifications to our change request involve any significant hazards consideration using the standards and criteria specified in 10 CFR 50.92. The Nuclear Regulatory Commission has provided guidelines concerning the application of these standards in 48 Federal Register 14870 which lists several categories of license amendments which are considered as very likely not to involve a significant hazards consideration. The proposed modifications to Technical Specification Change Request No. 88 have been reviewed and it is concluded that the modifications are administrative changes to the specifications and that the previous evaluation that the amendment does not involve any significant hazards consideration remains valid.

We are enclosing three signed originals and forty copies of this modification to our license amendments application. Please contact us if you have any questions concerning this submittal.

Very truly yours,



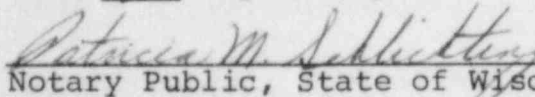
President

R. W. Britt

Enclosures

Copies to NRC Resident Inspector  
C. F. Riederer, PSCW

Subscribed and sworn to before me  
this 26th day of September 1984.

  
Notary Public, State of Wisconsin

My Commission expires May 4, 1986.