

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Washington Nuclear Plant - Unit 2

DOCKET NUMBER (2)
0 5 0 0 0 8 9 7 1 OF 0 2

TITLE (4)
RHR Isolation and Reactor Low Water Level Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
08	23	84	48	4	09	09	17	84			0 5 0 0 0 0
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THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9) 3	20.402(b)	20.406(e)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0,0,0	20.406(a)(1)(i)	50.38(e)(1)	50.73(a)(2)(v)	73.71(d)
	20.406(a)(1)(ii)	50.38(e)(2)	50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	50.72(b)(2)(ii)
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME
R. L. Koenigs, Compliance Engineer

TELEPHONE NUMBER
AREA CODE
5 0 9 3 7 7 7 - 1 2 5 0 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2279

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
B	B	O	-	-	-	-	-	-	N

SUPPLEMENTAL REPORT EXPECTED (14)
YES (if you complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

With the reactor shutdown, an RHR isolation and a reactor low water level trip (+13") occurred while placing the RHR system in the shutdown cooling mode.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		84	-0911	-010	012	OF	012

TEXT (If more space is required, use additional NRC Form 308A s/ (17))

Plant Conditions

- a) Power Level - 0%
- b) Plant Mode - 3

Event

With the reactor shutdown and at 60 psig, RHR Loop B was being warmed up for shutdown cooling (SDC) operation when RHR-V-8 shut automatically. The cause of this isolation was apparently high SDC suction flow. However no annunciation was provided for by design, and the isolation went unnoticed for approximately 15 minutes. During this time, the RHR Loop B partially drained (flowpath was to radwaste through the warmup line via RHR-V-40). When RHR-V-8 was reopened, a low reactor water level RPS actuation occurred.

When RHR-V-8 was reopened, reactor water began to fill the partially drained RHR loop. RHR-V-8 was immediately shut by the control room operator when reactor water level started to decrease at a rapid rate. The lowest reactor water level recorded was +10" during this event.

Immediate Corrective Action

After the reactor water level dropped to +10" and the shutdown cooling suction was isolated, a full inspection of the system was conducted to determine system leakage; none was found. After determining the cause of the level decrease the reactor level was increased to approximately +50" and the shutdown cooling flowpath was reestablished.

Future Corrective Action

A Plant Modification Record shall be prepared to provide control room audible annunciation on closure of valves within the Nuclear Steam Supply Shutoff System.

Safety Significance

This event did not compromise the health and safety of the public. All systems functioned as per design and an analysis of a similar postulated event (which bounds the event described here) indicates that level would not have decreased below 417". This is approximately 40" above low reactor water level 1(-129)". This analysis was originated as a result of our Operating Experience Review Program.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509)372-5000

Docket No. 50-397

September 17, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-091

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-091 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 0405 hours on August 23, 1984.

Very truly yours,

J. D. Martin
J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 84-091

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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