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VPNPD-92- 195 NRC-92-054

May 27, 1992

Document Control Desk
U.S. NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, D.C. 20555

Gentlemen:

DOCKETS 50-266 AND 50-301
GENERIC LETTER 88-20 (TAC NOS. 74452 A.D 74453)
REVISED SCHEDULE FOR SUBMITTAL OF IPE
POINT BEACH NUCLEAR PLANT UNITS 1 AND 2

On May 7, 1992, Wisconsin Electric Power Company personnel attended a meeting in Rockville, Maryland, with members of the NRC staff to discuss the progress on our programs to complete the Individual Plant Examination (IPE) for severe accident vulnerabilities required by Generic Letter 88-20. During the meeting, we discussed the background of PRA activities at Wisconsin Electric, our recent switch in contractors and software supporting the project, and the current status and schedule for submitting an IPE Summary Report to the NRC.

Our emphasis to the NRC staff was that our project changes were necessary to improve the plant-specific detail in our models and to more thoroughly address several technical details such as the two-unit interface and the human reliability analysis. We noted that we are planning to go well beyond the min.mum requirements for backend analysis by conducting our own severe accident thermal-hydraulic analysis using the MAAP code. Our goal has been to do more than just meet the minimum requirements of the generic letter for an IPE submittal. We plan to build a living PRA model which can be used as a management tool for plant decision making affecting modifications, operations, maintenance, safety evaluations, and possible plant life extension.

In our letter dated September 28, 1990, we had discussed our schedule for the IPE submittal and proposed a submittal date of December 31, 1992. In Mr. Samworth's letter dated March 18, 1991, this schedule was approved. As detailed in the May 7 meeting, we believe it is necessary for us to request a revised IPE submittal date of no later than June 1993. Although the majority of the work involved in the Level 1 and Level 2 PRA evaluations should be completed before the

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end of this year, we believe it is necessary to request this schedule extension in order to provide our staff sufficient time to conduct the second phase of independent review, evaluate the results of these analyses, and to plan and schedule any modifications or procedural changes which may be appropriate to address significant core damage sequences identified by the PRA. If the results of the PRA quantification demonstrate that only minimal changes to plant operations and procedures are required, it is likely that we would be able to submit our response to Generic Letter 88-20 well before June 1993. If we identify any significant severe accident vulnerability during the PRA quantification process, we shall communicate these results to the NRC staff before our formal submittal and initiate appropriate steps to address such a vulnerability or an expedited basis.

As suggested by the NRC staff, we agree that an additional meeting with NRC personnel some time in November 1992 is appropriate to discuss the results of the Level 1 and Level 2 PRA assessments. We shall contact you in fall to establish a firm date for that meeting.

We appreciated this opportunity to meet with the NRC staff and discuss our IPE progress. Should you have any outstanding questions as a result of the meeting or this letter, please let us know.

Sincerely,

Bob Link

Vice President Nuclear Power

Copy to: NRC Regional Administrator, Region III

NRC Resident Inspector