

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 2 4	PAGE (3) 1 OF 0 1
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TITLE (4)
Reactor Protection System Actuation During Unit 2 Refueling/Maintenance Outage

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																					
0 8	2 7	8 4	8 4	0 1 0	0 0	0 9	1 8	8 4			0 5 0 0 0																																																					
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) 5</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 0 0 0</td> <td>20.402(b)</td> <td></td> <td>20.406(c)</td> <td><input checked="" type="checkbox"/></td> <td>50.73(a)(2)(iv)</td> <td></td> <td>73.71(b)</td> </tr> <tr> <td>20.405(a)(1)(i)</td> <td></td> <td>50.36(e)(1)</td> <td></td> <td>50.73(a)(2)(v)</td> <td></td> <td>73.71(c)</td> </tr> <tr> <td>20.405(a)(1)(ii)</td> <td></td> <td>50.36(e)(2)</td> <td></td> <td>50.73(a)(2)(vii)</td> <td></td> <td rowspan="3">OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td>20.405(a)(1)(iii)</td> <td></td> <td>50.73(a)(2)(i)</td> <td></td> <td>50.73(a)(2)(viii)(A)</td> <td></td> </tr> <tr> <td>20.405(a)(1)(iv)</td> <td></td> <td>50.73(a)(2)(ii)</td> <td></td> <td>50.73(a)(2)(viii)(B)</td> <td></td> </tr> <tr> <td>20.405(a)(1)(v)</td> <td></td> <td>50.73(a)(2)(iii)</td> <td></td> <td>50.73(a)(2)(x)</td> <td></td> <td></td> </tr> </table>												OPERATING MODE (9) 5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											POWER LEVEL (10) 0 0 0	20.402(b)		20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)		73.71(b)	20.405(a)(1)(i)		50.36(e)(1)		50.73(a)(2)(v)		73.71(c)	20.405(a)(1)(ii)		50.36(e)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		
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LICENSEE CONTACT FOR THIS LER (12)

NAME M. J. Pastva, Jr., Regulatory Technician	TELEPHONE NUMBER 9 1 9 4 5 7 - 9 5 2 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X									

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 8-27-84, at 1032, during a Unit 2 refuel/maintenance outage, an automatic trip of both Reactor Protection System (RPS) Channels A and B occurred. At the time, the Unit 2 reactor control rods were inserted, reactor pressure was atmospheric, and reactor water temperature was 110°F. In addition, no Unit 2 Control Room alarm annunciations attributable to this event, other than the Reactor Auto Scram Channels A and B alarm, were received. Both RPS channels were reset shortly after the event.

A check of Unit 2 instrumentation, which may have initiated the event, did not reveal any abnormalities. It is felt the event may have resulted from a spurious upscale instrument spike of RPS instrumentation. This conceivably could have been caused by welding or mechanical shock during Unit 2 outage-related activities in the vicinity of RPS instrumentation. However, no correlation could be determined between the event and ongoing outage-related activities.

The anticipated decrease in Unit 2 refuel-related activities, as the Unit 2 refuel approaches completion, will help to reduce the possibility of inadvertent RPS actuations.

This event occurred in the most conservative plant condition.

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CP&L

Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

September 18, 1984

FILE: B09-13510C
SERIAL: BSEP/84-1977

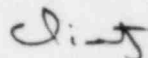
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U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-84-10

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,



C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/ag/LETAG

Enclosure

cc: Mr. R. C. DeYoung
Mr. J. P. O'Reilly

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